Relevant Nuclear Databases

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Utilized Databases

• ENDF/B

- The Evaluated Nuclear Data File begun in 1966, and run by the American lead Cross Section Evaluation Working Group (CSEWG).
- Decay Data Sublibrary contains spectrum data concerning nuclear decays for over 3800 materials (radioisotopes both in ground state and metastable isomers).
- Current published version: VIII.0 D.A. Brown, et al., Nucl.Data Sheets **148**, 1 (2018)
- Fission Yield Sublibrary provides independent and cumulative fission yields.
- Decay Data Sublibrary provides decay radiation spectra for all radiation types.



Image: CSEWG 2019 Group Photo (Last in-person meeting).







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Utilized Databases

• JEFF

- The Joint Evaluated Fission and Fusion File (JEFF) is an evaluated library produced via an European lead collaboration of Nuclear Energy Agency Data Bank participating countries.
- Current published version: 3.3 Plompen, A.J.M., Cabellos, O., De Saint Jean, C. *et al.* The joint evaluated fission and fusion nuclear data library, JEFF-3.3. Eur. Phys. J. A **56**, 181 (2020)
- JEFF library contains a decay data, and fission yield sublibraries.
- Provides us with Cumulative Fission Yield Data.







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Unutilized Sublibraries within Databases

ENDF/B and JEFF are used to calculate Electron and Neutrino Spectra via the Summation Method.

Summation Spectra = $\sum CFY_j(\sum I_{\beta jk}S_{jk})$



: Cumulative fission yield of a fission product.

: Beta decay intensity.

: Nuclear level to nuclear level antineutrino









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Unutilized Sublibraries within Databases

• JEFF Decay

- Does not include theoretical neutron rich nuclide decay spectra.
- Does not include beta intensities from recent TAGs experiments.
- ENDF/B Fission Yields
 - Contains anomalous cumulative yields values and uncertainties.

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Reliance on theory in ENDF/B-VIII.0 Decay Data

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Nucleus	$T_{1/2}$ (s)	$Q_{\beta-}$ (keV)	CFY	%total IBD
103 NL	1 5	5.022	2.165×10^{-2}	0.65
103 Zr	1.3	7,213	4.484×10^{-3}	0.36
$^{104}\mathrm{Nb}$	4.9	8,531	4.357×10^{-3}	0.32
^{101}Y	0.45	8,105	3.007×10^{-3}	0.31
104 m NI	0.94	8 531	3.331×10^{-3}	0.24
880	0.01	0,001	0.001×10^{-3}	0.21
о _Б е	1.0	0,052	2.710×10	0.21
142 Xe	1.2	5,285	6.886×10^{-3}	0.20
$^{94}\mathrm{Kr}$	0.21	$7,\!215$	1.802×10^{-3}	0.15
^{139}I	2.3	$7,\!174$	7.554×10^{-3}	0.15
$^{105}\mathrm{Nb}$	2.9	$7,\!422$	2.414×10^{-3}	0.14

TABLE I. The 12° largest contributors to the IBD antineutrino yield generated in the thermal neutron fission of 235 U for which only incomplete decay data is available, thus necessitating theoretical calculations to obtain the radiation spectra. The table includes their rounded values of half-lives, β -Q-values and cumulative fission yields, ordered by their percentile contribution to the total IBD yield.

R. Lorek, A.A. Sonzogni, A. Mattera and E.A. McCutchan, TBP.





ENDF/B-VIII.0 Fission Yield Anomalies





A.A. Sonzogni, E.A. McCutchan, T.D. Johnson, and P. Dimitriou, PRL 116, 132502 (2016)

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ENDF/B-VIII.0 Fission Yield Anomalies



Library	²³⁵ U	²³⁸ U	²³⁹ Pu	²⁴¹ Pu
JEFF-3.3	-173.155 ± 0.076	-173.047 ± 0.298	-173.585 ± 0.115	-173.523 ± 0.240
ENDF/B-VIII.0	-173.125 ± 0.943	-173.225 ± 0.585	-173.676 ± 0.417	-173.552 ± 0.410
ENDF/B-VIII.0 Mod.	-173.137 ± 0.039	-173.218 ± 0.081	-173.694 ± 0.073	-173.547 ± 0.092
Kopeikin [8] (England & Rider)	-173.43 ± 0.05	-173.39 ± 0.10	-173.87 ± 0.07	-173.82 ± 0.10

Table 1. *Y* and ΔY values, calculated with Eqn. 21.

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Anomalous large uncertainties for a number of stable or very long-lived fission products, for instance ⁹⁰Y and ⁹⁰Zr, resulting in large uncertainties in the parameter **Y** needed to calculate total energy released in fission.

$$Y = \sum C Y_k M_k,$$

A. Mattera and A.A. Sonzogni, BNL-220804-2021-INRE Report (2020).





<u>Updates</u>

Updates:

- ENDF/B-VII.0 contained no TAGS in β spectrums.
- Current version (VIII.0) released on February 2018
- Version VIII.1 in progress with ~435 materials updated so far.
- Updates are made as peer-reviewed published data becomes available.

NA22 is funding new fission yield sublibrary via a multi-lab effort. (PI: Toshihiko Kawano (LANL))





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Desired Improvements

In brief; the decay emission spectra is described as...

 $S(E_e) = NW(W^2-1)^{\frac{1}{2}}(W-W_o)^2 \times F(Z_k,W) \times C_L(W) \times C_{fs}(W) \times C_s(Z_k,W) \times C_{WM}(W) \times C_r(Zk,W) \times C_{exp}(W)$

Shape Factor represents an experimental correction and is dependent upon the chosen Fermi Function ($F(Z_k, W)$).

Shape Factor may have a significant impact on the antineutrino spectra. See, *First-forbidden transitions in reactor antineutrino spectra*, L. Hayen, *et al.*, PRC 99, 031301 (2019)

A new database format is needed to store the Shape Factor.

- If measured, must be stored.
- If stored, must fit within the current database.





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Desired Improvements

In order to properly quantify uncertainty in calculations cumulative fission yield correlations must be stored within database format.

 $\Delta^{2}S = \sum \Delta^{2}C_{j}S_{j} + C_{j}\Delta^{2}S_{j} + \sum S_{j}S_{k}\frac{Cor(C_{j}C_{k})}{\Delta C_{j}\Delta C_{k}}$

Plot illustrates independent and cumulative fission yield correlation matrices.









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Application to Antineutrino Spectra



IBD Yields as a function of prompt positron energy.

Summation from ENDF/B-VIII.1 decay data and JEFF-3.3 Cumulative Yields.

Daya Bay deduced antineutrino spectra for ²³⁵U and ²³⁹Pu.

Database Summation does not agree with deduced spectra.



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Application to ²³⁸U antineutrino spectrum



The new JEFF-3.3 fission yields produce a 238U antineutrino spectrum that is different from JEFF-3.1 and from the Mueller model.

Mueller model used an earlier version of JEFF.

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More TAGS data has become available since this earlier version.

Difference is based on fission yield data (decay data taken from ENDF/B-VIII.1).







Full current ENDF/B sublibraries available on Gitlab: <u>https://git.nndc.bnl.gov/</u> (Account required for access)





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