

Neutron capture spectra and other nuclear data plans



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- **Applications sometimes require accurate simulations of data**
 - Backgrounds
 - Energy conservation
 - Energy and multiplicity distributions per event
- **Very few experiments directly measure QC spectrum**
 - A lot of work to unfold detector response
 - Typically requires isotopically-enriched targets to interpret data
 - I'm still looking for good examples in the literature

Solution: Model QC spectra

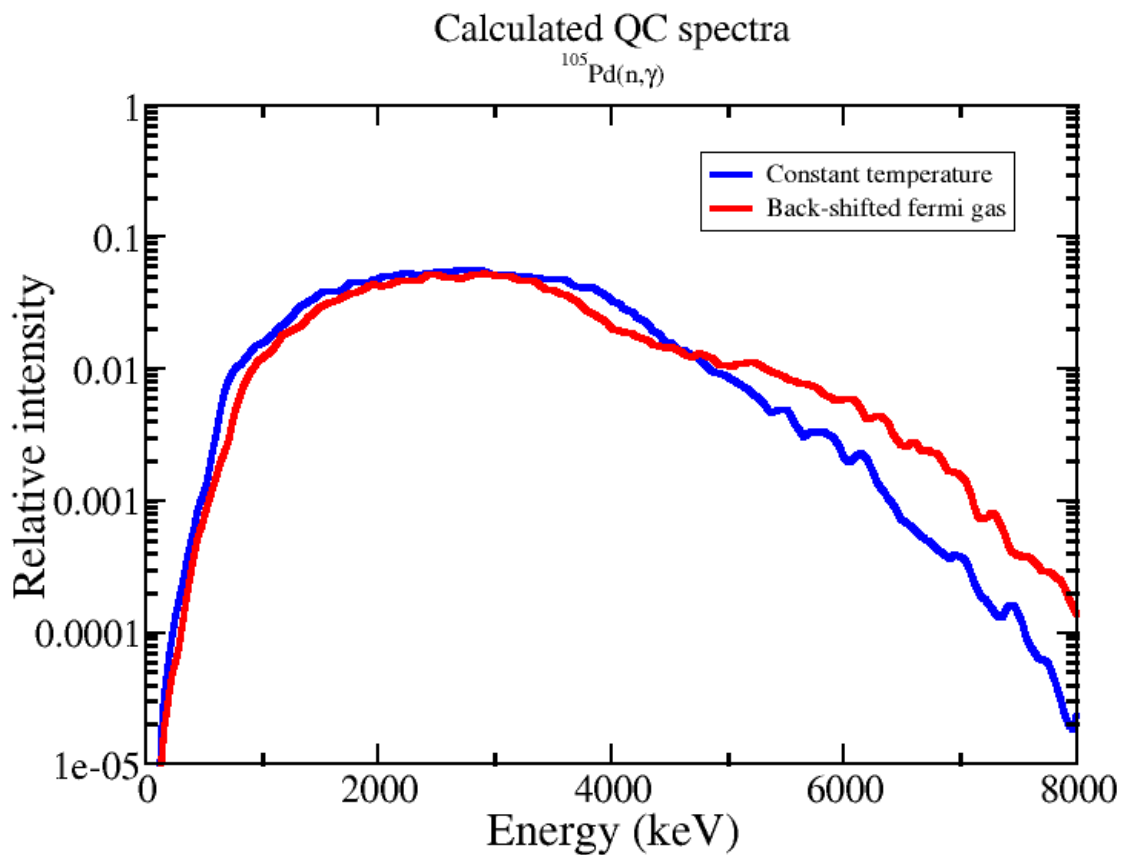
Problem: How do we validate calculations without direct experimental evidence?

The overall plan



- **Develop the γ -ray decay model**
 - γ -ray cascade algorithm from F. Becvar (NIM A417, 434 (1998))
 - Physics input from a variety of sources
- **Validate model against thermal neutron capture gamma-ray data**
 - Gamma-ray lines and relative intensities
 - » Institute of Isotope and Surface Chemistry, Budapest
 - » **EGAF**: Extensive re-evaluation of all the literature
 - Total partial widths: Γ_γ
- **Demonstrate that new model is predictive**
 - Nuclear spectroscopy: J^π assignments
 - Neutron capture cross sections from γ -ray cross sections
- **Crank out new evaluations for ENDF**
 - Database feeds transport codes such as MCNP

Sensitivity to level density explored



χ^2 for levels

- CT: 1.7
- BSFG: 1.7

Γ_γ Exp. 150(8) meV

- CT: ≈ 112 meV
- BSFG: ≈ 335 meV

Multiplicity

- CT: 4.3
- BSFG: 4.5

More data may be needed to zero in on QC spectrum

Light isotopes don't have a quasi-continuum



- **Converting EGAF into ENDF for $Z \approx < 20$**
- **Using current ENDF capture cross sections**
 - Recent data suggests changes for some isotopes
 - Multiplicity per cascade defined as

$$m_i = \frac{\sigma_i^{EGAF}}{\sigma_{(n,\gamma)}^{ENDF}}$$

- **One overall scale factor used to conserve energy**
 - Worst case so far is an $\approx 15\%$ correction

$$Q = A \sum_i m_i E_{\gamma_i}$$

Work by Brad Sleaford

11 isotopes converted and tested to date



Tgt	ZA	NK,#	linN.A.	(MugS(eV*Yield))	(Q-S(eV*Yield))	Comments/Status
1001	1	99.98%	2.2263E+06	-0.06%		Ace, Gendf Libraries complete
1002	1	100.00%	6.2502E+06	0.11%		Ace, Gendf Libraries complete
2003	1	0.00%	6.2502E+06	0.11%		
3006	3	0.07589	1.47E+07	-1.030000E-02		Ace, Gendf Libraries complete
3007	3	92.41%	2.0464E+06	-0.66%		Ace, Gendf Libraries complete
4009	12	1	7.02E+06	-3.001541E-02		Ace, Gendf Libraries complete
5010	10	19.82%	6.9402E+06			TBD
5011	9	0.8018	7.20E+06			TBD
6013	13		5.786693E+06	-0.169738		Natural
6012	6	98.89%	5.3922E+06	-9.01%		Ace, Gendf Libraries complete
6013	7	1.11%	6.9920E+06	14.49%		Ace, Gendf Libraries complete
7014	60	99.63%	1.0794E+07	0.34%		Ace, Gendf Libraries complete
8016	4	99.76%	4.2191E+06	-1.84%		Ace, Gendf Libraries complete
8017	20	0.03%	9.0172E+06	-12.06%		Natural
9019	165	100.00%	6.6043E+06	-0.05%		OK
10020	27	90.48%	6.7118E+06	0.73%		TBD
10021	11	0.27%	1.1456E+07	-10.54%		TBD
10022	10	9.25%	5.0577E+06	2.75%		TBD
11023	233	100.00%	6.5101E+06	6.46%		Libraries complete (natural)
12024	35	78.99%	7.3899E+06	-0.81%		In Process
12025	206	10.00%	1.0927E+07	1.49%		In Process
12026	41	11.01%	6.5472E+06	-1.61%		In Process
13027	230		7.55E+06	2.25%		Ace Gendf Libraries complete
14028	46	92.23%	1.7363E+07	-8.98%		Ace, Gendf Libraries complete
14029	98	4.68%	1.1288E+08			TBD
14030	38	3.09%	7.4716E+06			TBD
15031	158	100.00%	7.3497E+06			TBD
16032	101	95.02%	8.4862E+06			TBD
16033	249	0.75%	1.0342E+07			TBD

Example: $^{27}\text{Al}(n,\gamma)$ EGAF spectrum in ENDF format



ENDF/B- VI Evaluation, August 2004, R. F. Firestone (LBNL)	12 00 14 51	13
D. McNabb, B. W. Sl eafor d (LLNL)	120 0 1451	14
	1200 14 51	15
The prompt gamma-ray spectrum for thermal radiative capture 1	200 1451	16
(MF 12, MT 102) has been updated with new experimental data.[Fi]	1200 14 51	17
	1200 14 51	18
REFERENCE	12 00 14 51	21
	1200 14 51	22
[Fi] Data derived from the Evaluated Gamma-ray Activation File	1200 1451	23
(EGAF) as described in " Database of Prompt Gamma Rays from Slow	1200 14 51	24
Neutron Capture for Elemental Analysis", R.B. Firestone,	1200 1451	25
H.D. Choi, R. M. Lindstrom, G.L. Molnar, S.F. Mughabghab,	1200 1451	26
R. Paviotti-Corcuera, Zs. Revay, V. Zerkin, and C.M. Zhou,	1200 1 451	27
IAEA TECDOC, in press.	12 00 14 51	28
	1200 14 51	29
1 . 3027 00+4 2.674 975+1 1 0 291 013 25121 02275 19		
0 . 0000 00+0 0. 00000 0+0 0 0 1 171325 121022 7520		
17 2 13 25121 02275 21		
1 . 0000 00-5 2.230 130+0 1.00 0000+4 2.23 0130+ 0 1.0 00000 +5 2. 23000 0+013 25121 02275 22		
5 . 0000 00+5 2.450 000+0 1.00 0000+6 2.45 0000+ 0 2.0 00000 +6 2. 47000 0+013 25121 02275 23		
3 . 0000 00+6 2.500 000+0 4.00 0000+6 2.55 0000+ 0 6.0 00000 +6 2. 61000 0+013 25121 02275 24		
8 . 0000 00+6 2.840 000+0 1.00 0000+7 2.95 0000+ 0 1.2 00000 +7 2. 98000 0+013 25121 02275 25		
1 . 4000 00+7 3.050 000+0 1.70 0000+7 3.36 0000+ 0 2.0 00000 +7 3. 91000 0+013 25121 02275 26		
2 . 0000 01+7 0.000 000+0 1.50 0000+8 0.00 0000+ 0 13 25121 02275 27		
7 . 7240 26+6 0.000 000+0 0 2 1 613 25121 02275 28		
6 2 13 25121 02275 29		
1 . 0000 00-5 2.874 819-1 1.00 0000+4 2.87 4819- 1 1.0 00000 +5 0. 00000 0+013 25121 02275 30		
2 . 0000 00+7 0.000 000+0 2.00 0001+7 0.00 0000+ 0 1.5 00000 +8 0. 00000 0+013 25121 02275 31		

V&V: ^{27}Al irradiated with thermalized Cf neutrons



Outlook is good for delivering new thermal neutron capture spectra one year from now



- **Validating our model against data**
 - Experimental QC spectra would be useful
 - Γ_γ widths may be enough to select “best” model
- **Model is predictive**
- **Systematic uncertainties in the computed QC spectrum due to**
 - Level density
 - Strength functions
- **Need to add more test cases before confident in prescription**

Future plans for improving photon production

- **Non-thermal neutron capture spectra**
- **Inelastic scattering photon production**
- **Decay spectra**
- **Joint LBNL/LLNL website targeted to homeland security personnel interested in these issues**

Goal: Improve transport simulation capability for new radiochemical diagnostics (actinides)



- Add physics of low-energy neutron scattering to nuclear data code libraries to improve fidelity of (n, γ) simulations
 - MCAPM
 - NDF
- Collaborate with LANL on best estimates of actinide production/depletion cross sections with uncertainties
 - Good fission cross sections are key for fast neutrons
 - Good capture cross sections are key for slow neutrons

We bring an experimental focus to this collaboration

Fission and capture cross sections are hard to predict theoretically



$$\begin{aligned}\sigma_r &= \sigma_{(n,f)} + \sigma_{(n,n')} + \sigma_{(n,2n)} (+\sigma_{(n,\gamma)}) \\ &= \pi(R + D)^2 (1 - e^{-\alpha})\end{aligned}$$

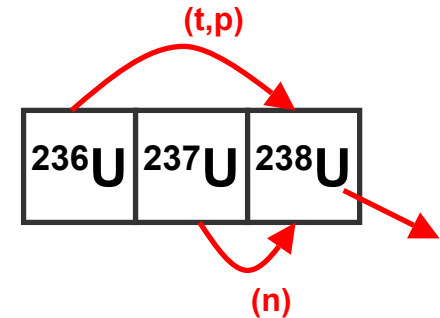
- **6-parameter fit with dependence on E_n , N , Z good to $\approx 1\%$**
 - 2 parameters for R
 - 4 parameters for α
- **σ_r for nuclei with no data to 3-4% for fast neutrons**
 - McNabb et al., submitted to NSE & ND conference
 - Follow up work in progress

**Fission cross sections are the missing link
Need good model of functional shapes of (n,n') and (n,2n)**

Two-prong effort on fission data



- Analyze old transfer data for $E < 2.2$ MeV
 - Extend to 14 MeV w/ theory
- Obtain new data with improved technology
 - Extend experiments to higher energies
 - Improve understanding of errors

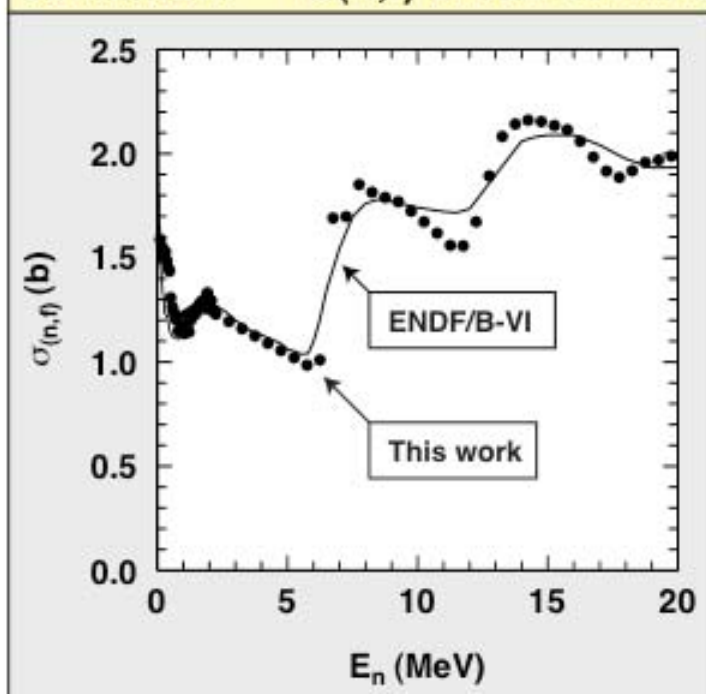


Both efforts use data from “surrogate” reactions

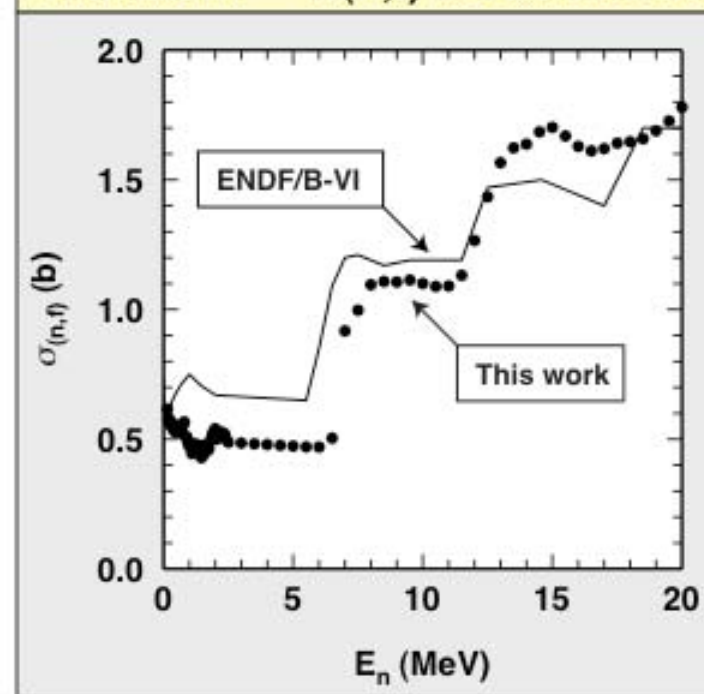
Preliminary extension of (n,f) to $E_n = 20$ MeV



Validation: $^{235}\text{U}(n,f)$ cross section



Prediction: $^{237}\text{U}(n,f)$ cross section



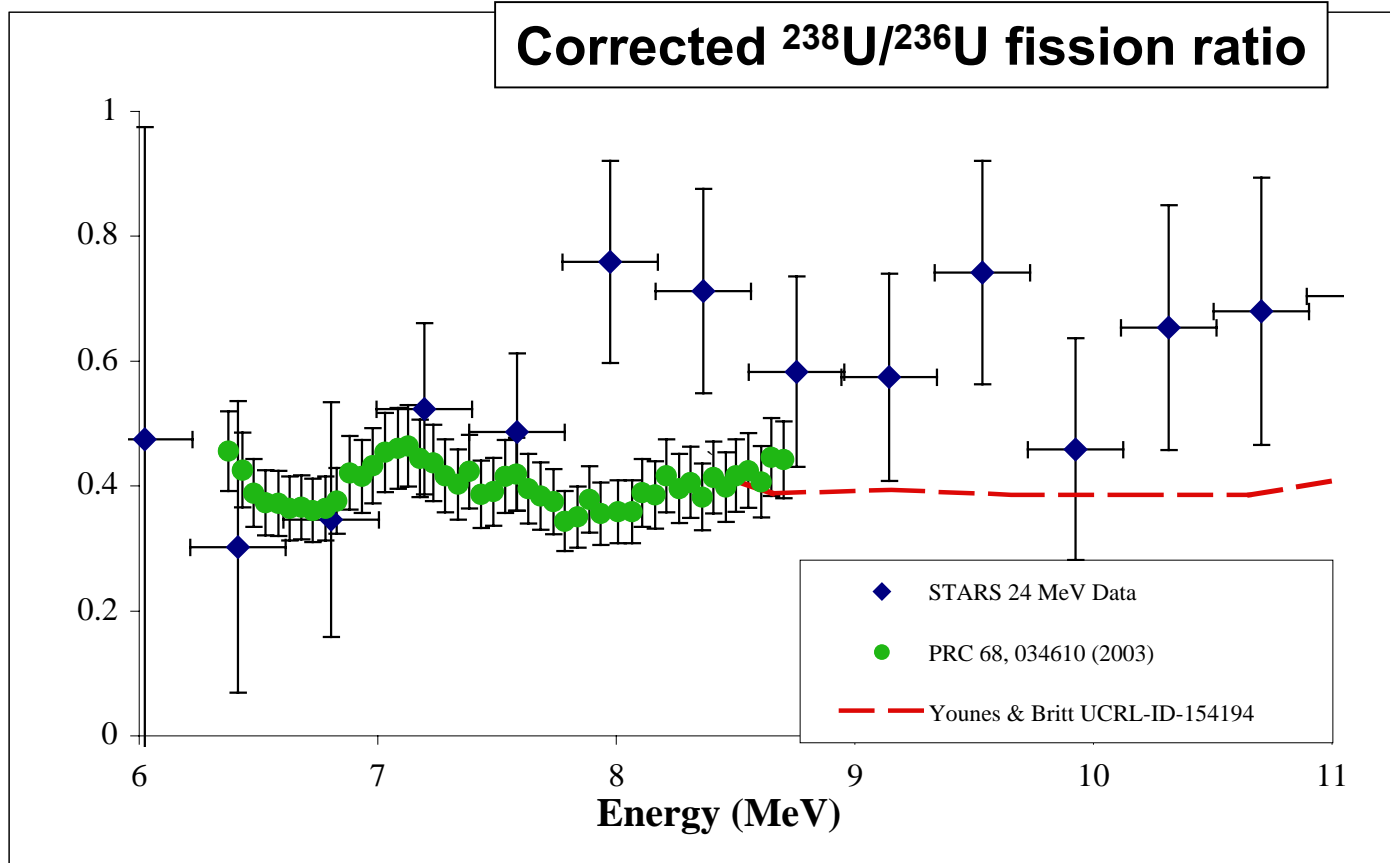
- Surrogate-data estimate of $^{235,237}\text{U}(n,f)$ currently limited to $E_n < 2.2$ MeV
 - – use linear extrapolation for 1st-chance fission
 - – use measured $A^{-1}\text{U}(n,f)$ for 2nd+3rd-chance fission
- W. Younes et al., UCRL-ID-154194 (2003)

Looks promising, but **need improved fission model**
⇒ predictions up to 20 MeV entirely from surrogate data

Validation of the surrogate technique for Actinides: $^{236,238}\text{U}(d,d')$ with STARS - May 2004



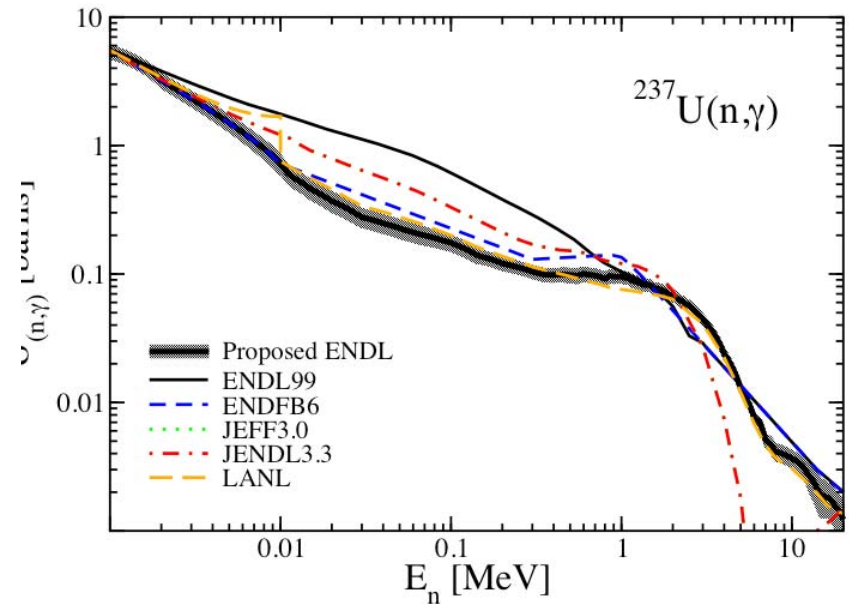
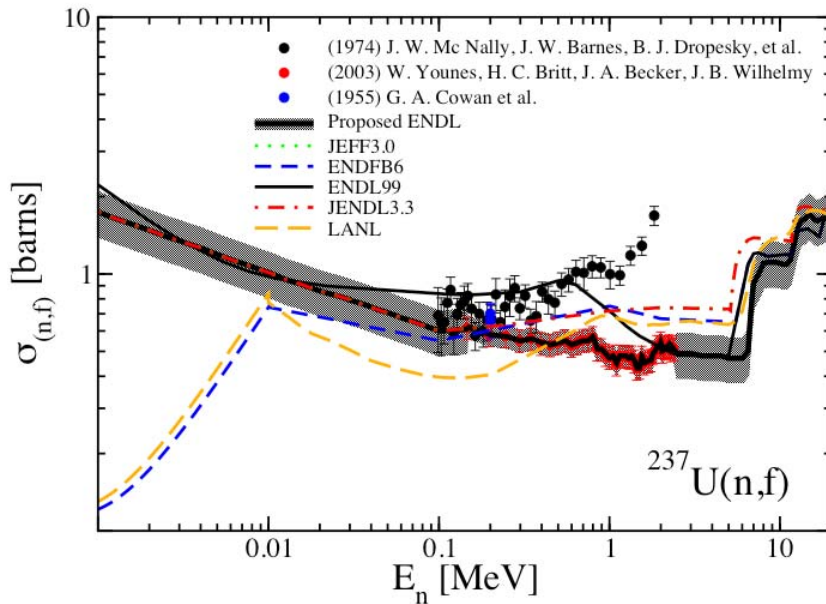
Work by L. Bernstein, J. Church, L. Ahle, J. Punyon



**PRELIMINARY ratios in agreement with W. Younes et. al.
 ^{238}U analysis (non-ratio) still in progress**



Generating the starting point for a global fit



First-pass results for $^{232-241}\text{U}$ cross sections delivered:

- Adopted model calculations from Japan, Europe, US
- Modified results in key areas based on data, sum rules
- Estimated model uncertainties in a crude fashion

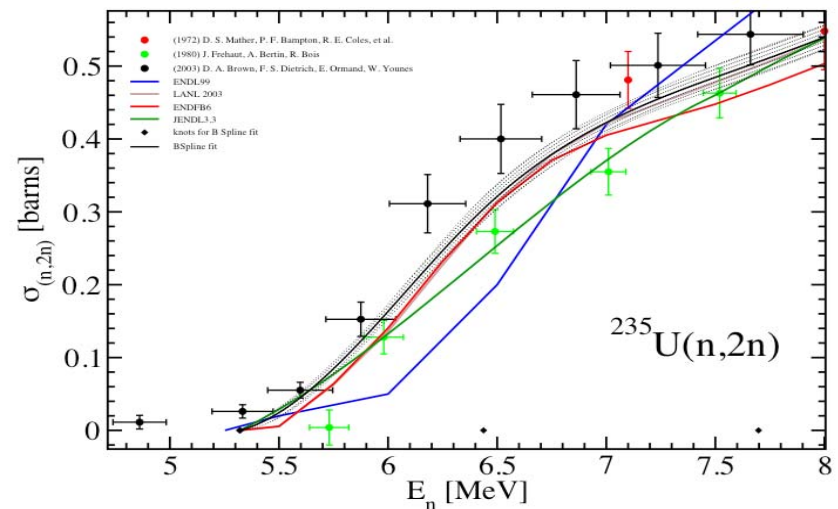
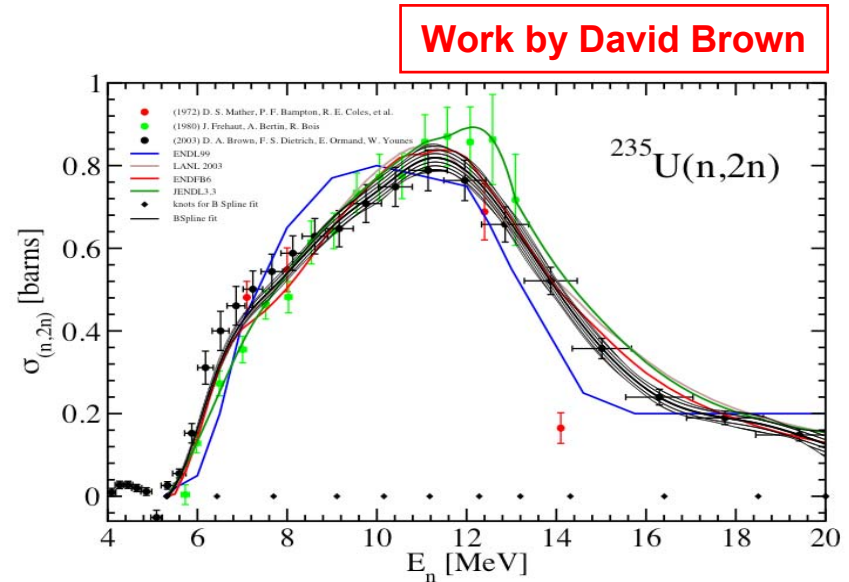
“I’m pleased to see that uncertainties are being added to the data.”

-- C. McMillan. B-Div Leader

Constrained fits to evaluated data, uncertainties



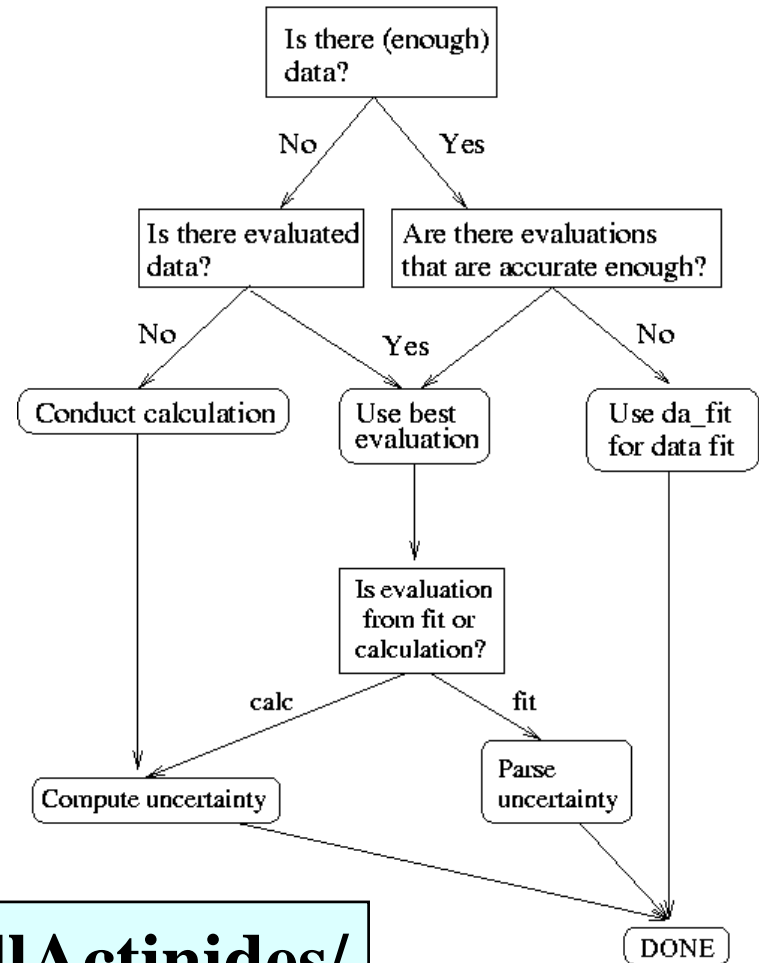
- **Production and depletion of actinide isotopes**
 - Forensic signatures
 - Uncertainties required
- **Simultaneous fit to actinide cross section data**
 - Data-driven
 - » Covariances included
 - Theoretical assumptions
 - » Explicit constraints
 - Uncertainties intrinsic



Automating as much as we can to generate first-pass estimate for all nuclei



- *x4i* to parse exp. data
- *fete* to parse ENDF/B data
 - prelim. ENDF/B-6, JEFF3.0, JENDL3.3
- Constrained, generalized least-square spline fitter



<http://nuclear.llnl.gov/CNP/allActinides/>

New issues & future directions



- **We're close to having a first-pass cross section set**
 - Now our goal is to “do it right” (global fit)
- **We are excited about surrogate fission effort**
 - Will require a sustained multi-year effort
- **Working toward more involved in DANCE (n,γ) measurements**

NADS 1.4 & 1.5 (Changes from last year)



- **Rewrote 2-D plotting**
 - Publication quality graphics
 - More control over look and feel (similar to xmgrace)
 - Faster plotting with large data sets
 - Plots uncertainties
 - Preferences can be saved
- **Added EXFOR cross section data**
- **Improved computational features**
 - Merge EXFOR data sets
 - Save/load computation sessions
 - Commands history
 - Some 2-D data (vector) math added
- **New table features**
 - Tab-delimited text file may be read in
 - Tables can be edited and saved or replotted
- **Modified server start-up procedure**
 - Starts up in 30 seconds, loads data into memory as requested
 - Will help us deal with lost port problem / crashes