## Covariance Data for JENDL-3.3 Actinides

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## Actinide Nuclides with Covariance Data

#### • JENDL-3.3

U-233, U-235, U-238, Pu-239, Pu-240, Pu-241

The data of these nuclides were released together with covariance data.

#### Additional nuclides

Np-237, Pu-242, Am-241, Am242m, Am-243, Cm-244 Covariance data were evaluated after release of JENDL-3.3.

## Covariance Data for Additional Actinides

- Fission cross section Experimental data were analyzed with a least-squares fitting code GMA developed by Poenitz.
- Capture cross section Statistical model code CASTHY and covariance generation code KALMAN developed by Kawano were used. Covariance matrices were calculated from sensitivities and uncertainties of model parameters.
- Resonance parameters Standard deviations were given to a resonance energy, neutron, capture and fission widths of each resonance.
- Number of neutrons per fission Experimental data were fitted with a straight line.

#### **Examples of fission cross sections**





Uncertainties are smaller than 3 % in this energy range.



evaluation.



Large cross section groups were ignored in the evaluation.



Covariance data file was processed with ERRORJ code. This figure was written from the ERRORJ output.

#### **Examples of capture cross sections**



#### **Covariance of Am-241 capture**





No experimental data are available. KALMAN calculation was done by assuming uncertainty of 20% at 30 keV.



#### **Covariance of Am-243 capture**



# Examples of number of neutrons per fission





### **Nuclides with Covariance Data**

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Covariance data have been prepared for 32 nuclides of JENDL-3.3.

• JENDL-3.3 (20)

H-1, B-10, B-11, O-16, Na-23, Ti-48, V, Cr-52, Mn-55, De-56, Co-59, Ni-58, Ni-60, Zr-90, U-233, U-235, U-238, Pu-239, Pu-240, Pu-241

 Additional nuclides (12)
N-15, Pb-206, Pb-207, Pb-208, Bi-209, Np-237, Pu-238, Pu-242, Am-241, Am242m, Am-243, Cm-244