Uncertainty Management for Nuclear Systems Simulation

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Presented at the Neutron Cross-Sections Covariance Workshop, Port Jefferson, NY, June 24-27, 2008





Acknowledgement and Research Team

□ Sponsors:

- Electric Power Research Center of NCSU, Fellowship Naval Nuclear Propulsion Fellowship Program, INL LDRD, DOE NERI, GE-Hitachi
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Introduction

- Recently, modeling and simulation (M&S) recognized as viable tools to achieve optimal design and operation of existing and next generation reactor systems (Gen-IV)
- Design and evaluation strategies projected to reduce reliance on expensive validating experiments and employ accurate M&S as primary design and analysis tool
- M&S must have uncertainty management framework
 - Quantifiable error bounds on simulation results
 - Means to understand various sources of errors
 - Mean to reduce identified sources of errors
 - Means to integrate experiments, and devise their optimal design



Neutron Cross-Section

Many studies proved that nuclear data uncertainties constitute major source of errors in neutronics design calculations

Resonance Parameter Uncertainty leads to 0.15% uncertainty in EOC k-effective (\$600K in FCC)





Importance of Uncertainty Management

- Define required system design margins
- Identify key input data and associated models contributing most to quantified uncertainties
- Alter design to make it less sensitive to identified key sources of uncertainties
- Optimize experiments design to reduce uncertainties
- Increase design freedom by reducing design margins realized by higher fidelity calculations
- These goals to be achieved via simulation to minimize reliance on expensive experiments

Definitions

Consider a computational model describing an engineering system:

$$\bar{y} = \Theta(\bar{x})$$

- **Sensitivity:** Rate of Change of output with respect to input
- Uncertainty: Confidence in calculated results
- Data Assimilation: Reduction of calculations uncertainties



Sensitivity Analysis Goal

□ Given a system model:

 $\vec{y} = \vec{\Theta}(\vec{x})$

where $\bar{x} \in \square^n$ are input data (physical constants, operating conditions, control parameters, etc.), and $\bar{y} \in \square^m$ are output responses (system attributes of interest to design, operation, and safety)

Calculate <u>at a minimum</u> first order derivatives of output responses with respect to input data

$$\boldsymbol{\Theta}_{ij} = \frac{\partial y_i}{\partial x_j}, i = 1, \dots, m, j = 1, \dots, n$$

Uncertainty Analysis Goal

- Given system model and input data uncertainties calculate output responses uncertainties.
 <u>Need:</u> Sensitivity Analysis
- □ Data uncertainties described <u>at a minimum</u> by probability distributions' means and standard deviations $\vec{x}_0 = \mathbb{E}[(\vec{x})]$ $\mathbb{E}[(\vec{x} - \vec{x}_0)^T]$

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Data Assimilation Goal

- Given measured system responses, adapt model to increase simulation fidelity by accounting for:
 - Modeling errors due to simplifying assumptions (Unclear how to accomplish?)
 - Numerical errors due to discretization (Emerging posterior and goal-oriented techniques)
 - Boundary Conditions characterizing interaction between various modeling stages: (calls for rigorous approaches)

Input data errors

(well-established approaches: requires model inversion, sensitivity analysis, and input data uncertainties)

$$\min_{x} \left\| \bar{y}^{m} - \bar{\Theta}(\bar{x} + \delta \bar{x}) \right\|^{2} + \alpha^{2} \left\| \delta \bar{x} \right\|^{2}$$

Uncertainty Management Steps Linear Approximation

Evaluate sensitivity information

$$\boldsymbol{\Theta}_{ij} = \frac{\partial y_i}{\partial x_j}, i = 1, \dots, m, j = 1, \dots, n \Rightarrow \boldsymbol{\Theta} = \begin{bmatrix} \boldsymbol{\Theta}_{11} & \boldsymbol{\Theta}_{1n} \\ \vdots & \vdots \\ \boldsymbol{\Theta}_{m1} & \vdots & \boldsymbol{\Theta}_{mn} \end{bmatrix}$$

Obtain input data covariance matrix

$$\mathbf{C}_{x} = E\left[(\vec{x} - \vec{x}_{0})(\vec{x} - \vec{x}_{0})^{T}\right]$$

Calculate of output data covariance matrix

$$\mathbf{C}_{y} = E\left[(\vec{y} - \vec{y}_{0})(\vec{y} - \vec{y}_{0})^{T}\right] = \mathbf{\Theta}\mathbf{C}_{x}\mathbf{\Theta}^{T}$$

Identify key sources of errors:

 $\left(\mathbf{C}_{x}+\mathbf{\Theta}^{T}\mathbf{\Theta}\right)^{-1}$

Why UQ Challenging? Example: Nuclear Reactors Modeling

- Fully resolved description of reactor is not practical even with anticipated growth in computer power over foreseeable future
- Multi-level homogenization theory adopted to render reactor calculations in practical run times with reasonable accuracy
- Input data: cross-sections, design data, etc.
- Output data: criticality, power, thermal margins, reactivity coefficients, etc.



Assemblies are combined to create the reactor core



Spatial Heterogeneity of nuclear reactor core

BWR Example (Size of I/O streams)



Sensitivity Forward Approach $\Theta \delta \vec{x} \square \vec{\Theta} (\vec{x}_0 + \delta \vec{x}) - \vec{\Theta} (\vec{x}_0)$

- Perturb input data one-at-a-time to calculate sensitivities of all outputs with respect to the perturbed input
- Suited for problems with few inputs and many outputs
- Variations:
 - Simultaneously perturb all inputs based on their prior PDFs; repeat until the output PDFs converge
 - Suitable for non-Gaussian distributions, and nonlinear systems.

Difficult to infer sensitivity information

 $\frac{\partial y_1}{\partial x_r}$ ∂x_1 $\frac{\partial y_2}{\partial x_1} \quad \frac{\partial y_2}{\partial x_r}$ $\frac{\partial y_m}{\partial x_r}$

Sensitivity Reverse Approach

 $\mathbf{\Theta}^{T} \delta \, \overline{y} \, \Box \, ???$

- Generalized Perturbation Theory
 - Based on select output response, constructs adjoint model to calculate the response sensitivities with respect to all input data
 - Suited for problems with many inputs and few outputs
 - Difficult to implement for legacy codes

$$\begin{bmatrix} \frac{\partial y_1}{\partial x_1} & \frac{\partial y_1}{\partial x_2} & \cdots & \frac{\partial y_1}{\partial x_n} \\ \frac{\partial y_r}{\partial x_1} & \frac{\partial y_r}{\partial x_2} & \cdots & \frac{\partial y_r}{\partial x_n} \end{bmatrix}$$

- Replace original I/O streams by mathematical subspaces
- Subspaces are mathematical abstractions denoting change of basis in the I/O streams:
 - Create new I/O variables (called active DOFs).
 - Dimensions of subspaces are much smaller than original I/O streams
 - Each variable (active DOF) is a linear combination of all original variables, with weights reflecting importance of original variables
 - Subspaces identified by means of stochastic approach involving randomized matrix-vector and matrix-transpose-vector products
 - Mathematically, this process is equivalent to finding rank revealing decomposition of sensitivity and uncertainty matrices
 - Requirement: matrices be ill-conditioned

Singular Values Spectrum

Singular Value Triplet Index

Philosophy of Subspace Methods

- In Euclidean sense, one can change n inputs to a computational model in n different ways, however, for most complex codes, only a subset r<<n leads to noticeable changes in outputs.</p>
- Active Degrees of Freedom denote the various changes in inputs leading to changes in outputs.
- Most outputs of interest to designers and operators are often integral quantities, e.g. power, reactivity, thermal margins, etc., (hence dimensionality reduction)

Active and Inactive DOFs: Example

Consider a simple model with one output response (energy produced from fission) and *n* input data (fission cross-sections of *n* different isotopes)

$$E = \sum_{i=1}^{n} \kappa_{i} N_{i} \sigma_{i} \Phi = \overline{\vartheta}^{T} \overline{\sigma}$$
$$\vartheta_{i} = \frac{\delta E}{\delta \sigma_{i}} = \kappa_{i} N_{i} \Phi$$

Consider inverse problem: How to select $\bar{\sigma}$ for some *E*? $\sigma_{2} \quad \vec{\sigma} = \vec{\sigma}^{\text{active}} + \vec{\sigma}^{\text{inactive}}$ Inactive DOF \vec{g} Active DOF

 $\delta \vec{\sigma} \propto \vec{\sigma}^{
m active}$

Background for Subspace Methods

Dimensionality reduction induced by a multi-level homogenization-type model can be described by Fredholm integral Equation of the first kind

$$\delta y(\varpi) = \int \Theta(\varpi, t) \Box \delta x(t) dt$$

Every square-integrable *kernel* has mean convergent singular value expansion of the form (Schmidt 1907-1908):

$$\vartheta(\varpi, t) = \sum_{i=1}^{\infty} \mu_i u_i(\varpi) \upsilon_i(t)$$

Singular Value Decomposition (SVD) is the algebraic version of SVE (Eckart and Young 1936-1939):

$$\mathbf{\Theta}^{m \times n} = \sum_{i=1}^{r} s_{i} \vec{u}_{i} \vec{\upsilon}_{i}^{T} = \mathbf{U}_{m \times r} \mathbf{\Sigma}_{r \times r} \mathbf{V}_{n \times r}^{T}$$

Consider a multi-level model composed of **k** sub-models (also applies to various components of a single sub-model):

1. Forward Runs:

2. Reverse Runs (ex. Adjoint):

$$\boldsymbol{\Theta}^* = \boldsymbol{\Theta}_k^* \leftarrow \boldsymbol{\Theta}_{k-1}^* \leftarrow \boldsymbol{\Theta}_2^* \leftarrow \boldsymbol{\Theta}_1^* \models r_{\boldsymbol{\Theta}}$$

-Reverse model runs only r_{Θ} times, i.e. rank of overall model. -Reverse runs only required for rank-deficient sub-models.

Q: What if reverse model infeasible for a sub-model or component? **A:** Given input subspace of dimension r_0 , run forward model r_0 times, and via a RVD, reduce the input subspace to r_{e}

Example 1: Boiling Water Reactor

- Part of GE-Hitachi funded research on 'Development of Adaptive Simulation Algorithms for BWRs'
- Given voluminous amount of data routinely collected from operating nuclear power plants, and maturity of neutronics calculations over past five decades, can one use a data assimilation to enhance agreement between measurements and predictions by adjusting cross-sections?

BWR Case Study

AMPX – ORNL ENDF Processing Code System

- Processes ENDF covariance data into 44 group energy structure
- SCALE5.0 libraries (PUFF3)
 - 44GROUPV5COV 29 isotopes including H, B, Al, U, Pu, and Minor Actinides et al.
 - 2. 44GROUPANLCOV 30 additional isotopes including Gd, Sm, Zr, et al.
- SCALE5.1 libraries Evaluations for V5 and V6 covariance data
- **TRITON -** ORNL lattice physics code
 - GE14 10x10 lattice design

BWR: Few-Group Cross-section Uncertainties

BWR: Power Distribution Uncertainties

BWR: Data Assimilation "Virtual Approach"

BWR: Data Assimilation "Virtual Approach"

BWR: I/O Streams SVDs

Example 2: Sodium Fast Reactor

- Work part of NERI on 'Management of Data Uncertainties and Optimum Design of Experiments for Gen-IV systems'
- Selected for analysis: ABTR core + ZPR experiments
- Research requires following capabilities:
 - Availability of group x-section uncertainties
 - Propagation of group x-section uncertainties to ABTR key attributes uncertainties
 - Data assimilation for x-sections using ZPR measurements
 - Reevaluation of ABTR's uncertainties using adjusted x-sections

ABTR: I/O Streams SVD

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Results: Relative Reaction Rate Uncertainty Data

Relative Reaction Rate Uncertainty

Conclusions

- For multi-level models exhibiting reduction in dimensionality through various levels, significant computational savings are possible via a subspace approach
- Only information belonging to '**active**' subspaces are communicated between various levels.
- Reverse models only required for the rank-deficient submodels, thus relaxing need for full adjoint capability, which can be quite challenging for linked code system.
- If reverse model infeasible, use a two-step reduction process to identify the active I/O subspace.
- Result is a framework for uncertainty management that can be applied effectively on a routine basis

Future Work

- Extend methodology to adjust resonance parameters directly using a probabilistic Monte Carlo model
- Develop methodology to situations when nonlinear behavior must be considered
 - Weak nonlinearities: Guide deterministic calculations for second order derivatives, i.e. Hessian operators, using active DOFs (r) from linear model (Computational cost ~ r^2)
 - Strong nonlinearities: Hybrid deterministic-probabilistic approach to bias stochastic samples using active DOFs from linearized model
 - Implicit assumption of these developments is that higher order derivatives may be ignored if first order derivatives are small

Questions?

