COMMARA-3

Processed ENDF/B-VII.1 covariance library

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Commara 2.0 materials

- 110 materials most relevant to fast reactor R&D
  - 12 light nuclei (LANL)
  - 78 structural materials (BNL)
  - 20 major and minor actinides (LANL + BNL)

- 135 files
  - 110 cross section covariances,
  - 20 nubars,
  - 3 PFNS,
  - 2 mubars

^{1}H	^{28}Si	^{92}Mo	^{109}Ag	^{149}Sm	^{232}Th
^{2}H	^{29}Si	^{94}Mo	^{127}I	^{151}Sm	^{233}U
^{4}He	^{30}Si	^{95}Mo	^{129}I	^{152}Sm	^{234}U
^{6}Li	^{50}Cr	^{96}Mo	^{131}Xe	^{153}Eu	^{235}U
^{7}Li	^{52}Cr	^{97}Mo	^{132}Xe	^{155}Eu	^{236}U
^{9}Be	^{53}Cr	^{98}Mo	^{134}Xe	^{155}Gd	^{238}U
^{10}B	^{55}Mn	^{100}Mo	^{133}Cs	^{156}Gd	^{237}Np
^{11}B	^{54}Fe	^{99}Tc	^{135}Cs	^{157}Gd	^{238}Pu
^{12}C	^{56}Fe	^{101}Ru	^{139}La	^{158}Gd	^{239}Pu
^{15}N	^{57}Fe	^{102}Ru	^{141}Ce	^{160}Gd	^{240}Pu
^{16}O	^{58}Ni	^{103}Ru	^{141}Pr	^{166}Er	^{241}Pu
^{19}F	^{60}Ni	^{104}Ru	^{143}Nd	^{167}Er	^{242}Pu
^{23}Na	^{90}Zr	^{106}Ru	^{145}Nd	^{168}Er	^{241}Am
^{24}Mg	^{91}Zr	^{103}Rh	^{146}Nd	^{170}Er	^{242m}Am
^{25}Mg	^{92}Zr	^{105}Pd	^{148}Nd	^{204}Pb	^{243}Am
^{26}Mg	^{93}Zr	^{106}Pd	^{147}Pm	^{206}Pb	^{242}Cm
^{27}Al	^{94}Zr	^{107}Pd		^{207}Pb	^{243}Cm
	^{95}Zr	^{108}Pd		^{208}Pb	^{244}Cm
	^{96}Zr			^{209}Bi	^{245}Cm
	^{93}Nb				^{246}Cm

ENDF/B VII.1 covariance materials

184 materials: 12 Light, 99 structural, 73 Actinides

- ^{1}H, ^{2}H, ^{4}He, ^{6}Li ^{7}Li, ^{9}Be, ^{10}B, ^{11}B, ^{0}C, ^{15}N, ^{16}O, ^{19}F, ^{24}Mg, ^{25}Mg, ^{26}Mg, ^{27}Al, ^{28}Si, ^{29}Si, ^{30}Si, ^{35}Cl, ^{37}Cl, ^{39}K, ^{41}K, ^{46}Ti, ^{47}Ti, ^{48}Ti, ^{49}Ti, ^{50}Ti, ^{50}Cr, ^{52}Cr, ^{53}Cr, ^{54}Cr, ^{55}Mn, ^{54}Fe, ^{56}Fe, ^{57}Fe, ^{59}Co, ^{58}Ni, ^{60}Ni, ^{89}Y, ^{90}Zr, ^{91}Zr, ^{92}Zr, ^{93}Zr, ^{94}Zr, ^{95}Zr, ^{96}Zr, ^{95}Nb, ^{92}Mo, ^{94}Mo, ^{95}Mo, ^{96}Mo, ^{97}Mo, ^{98}Mo, ^{100}Mo, ^{99}Tc, ^{101}Ru, ^{102}Ru, ^{103}Ru, ^{104}Ru, ^{106}Ru, ^{105}Pd, ^{107}Pd, ^{108}Pd, ^{109}Ag, ^{127}I, ^{129}I, ^{131}Xe, ^{132}Xe, ^{134}Xe, ^{133}Cs, ^{135}Cs, ^{139}La, ^{141}Ce, ^{141}Pr, ^{143}Nd, ^{145}Nd, ^{146}Nd, ^{148}Nd, ^{147}Pm, ^{149}Sm, ^{151}Sm, ^{142}Sm, ^{153}Eu, ^{155}Eu, ^{152}Gd, ^{153}Gd, ^{154}Gd, ^{155}Gd, ^{156}Gd, ^{157}Gd, ^{158}Gd, ^{160}Gd
- ^{166}Er, ^{167}Er, ^{168}Er, ^{170}Er, ^{180}W, ^{182}W, ^{183}W, ^{184}W, ^{186}W, ^{191}Ir, ^{193}Ir, ^{197}Au, ^{204}Pb, ^{206}Pb, ^{207}Pb, ^{208}Pb, ^{209}Bi, ^{225}Ac, ^{226}Ac, ^{227}Ac, ^{227}Th, ^{229}Th, ^{230}Th, ^{231}Th, ^{232}Th, ^{233}Th, ^{234}Th, ^{229}Pa, ^{230}Pa, ^{232}Pa, ^{230}U, ^{231}U, ^{232}U, ^{233}U, ^{234}U, ^{235}U, ^{236}U, ^{238}U, ^{234}Np, ^{235}Np, ^{236}Np, ^{237}Np, ^{238}Np, ^{239}Np, ^{236}Pu, ^{237}Pu, ^{238}Pu, ^{239}Pu, ^{240}Pu, ^{241}Pu, ^{242}Pu, ^{244}Pu, ^{246}Pu, ^{240}Am, ^{241}Am, ^{242m1}Am, ^{243}Am, ^{240}Cm, ^{241}Cm, ^{242}Cm, ^{243}Cm, ^{244}Cm, ^{245}Cm ^{246}Cm, ^{248}Cm, ^{249}Cm, ^{250}Cm, ^{245}Bk, ^{246}Bk, ^{247}Bk, ^{248}Bk, ^{250}Bk, ^{246}Cf, ^{249}Cf, ^{250}Cf, ^{251}Cf, ^{252}Cf, ^{253}Cf, ^{254}Cf, ^{251}Es, ^{252}Es, ^{253}Es, ^{254}Es, ^{254m1}Es, ^{255}Es, ^{255}Fm

Scope

- Associated with neutron cross sections from ENDF/B-VII.1
- Multigroup (processed) covariances with uniform lethargy energy groups (33 groups, flux: thermal–1/E–fiss spec)
- Reaction channels:
  - (n,el), (n,inl), (n,2n), (n,γ)
  - (n,f) & nubar MF31 (73)
  - prompt fission neutron spectra (PFNS, ^{238,239,240}Pu LANL, JENDL-4 MF35 (85)
  - Mubars MF34 (116 & additional from JENDL)

Methodology

- Covariance evaluation methodology determined by priorities:
  - Most important materials treated individually
  - Medium importance materials treated with simplified methods
  - Low priority materials (mostly fission products) treated with low-fidelity type approach

Methodology

Thermal and Resonance Region

- Source of data
- Experiments
- ENDF file (retroactive method)
- Atlas of Neutron Resonances (ANR)
- SAMMY analysis
- full analysis (MF32, Exp. data)
- retroactive (MF32, ENDF file)
- EMPIRE Resonance Module (MF32, ANR, scattering radius and thermal point uncertainties reproduced through correlations (if possible)
- “Kernel Approximation” (MF33, ANR)
- MF32 with systematic uncertainties in MF33
- ‘low-fidelity’ (Mark Williams) solution
- Assimilation

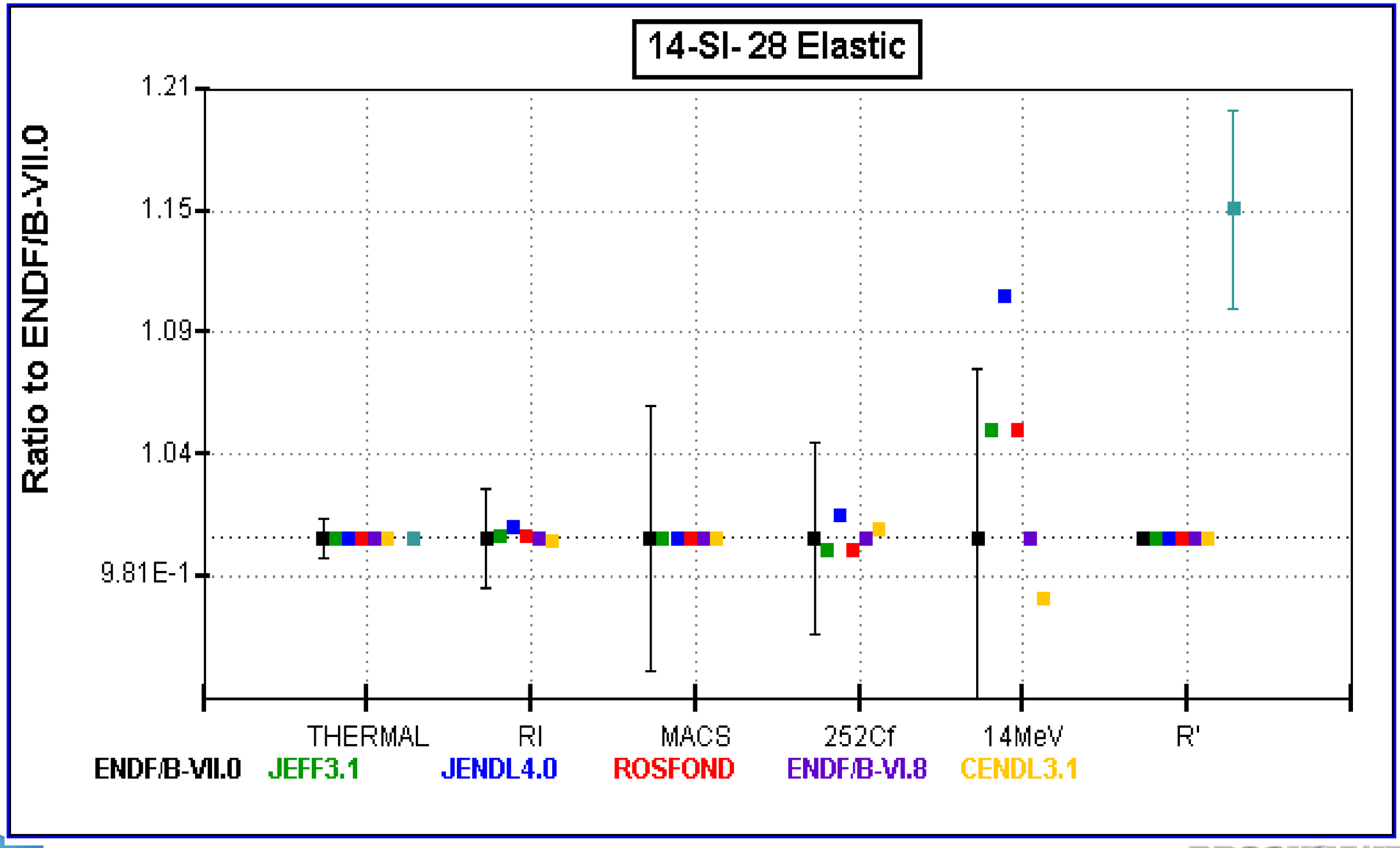
Fast neutron range (MF33)

- EMPIRE/KALMAN considering experimental data
- Least Square fitting of experimental data (SOK code)
- EMPIRE/KALMAN without experimental data (Low-Fidelity)
- Dispersion analysis - differences among evaluations (and exp. data)
- Reconsider previous work (ENDF/B-VI.8, Low-Fidelity)
- Visual analysis of experimental data
- Assimilation

Quality Assurance

- New web-based Sigma-QA (A. Sonzogni) allows visual and also quantitative inspection of:
  - Differential uncertainties (dynamic)
  - Integral uncertanties (static)
- UnCor applied to full library, performs 8 tests, warnings for possible problems including:
  - small uncertainties: (n,tot)<1%, (n,el) and (n,γ)<2%, etc.
  - non-positive-definite matrices (fixable for all but PFNS)
  - PFNS covariance not summing to zero (not usually a problem)
- non-positive-definite matrices are usually fixable by slightly reducing the off-diagonal elements. If not, more drastic measures may be required.

^{28}Si integral quantities from Sigma-QA
(A. Sonzogni)



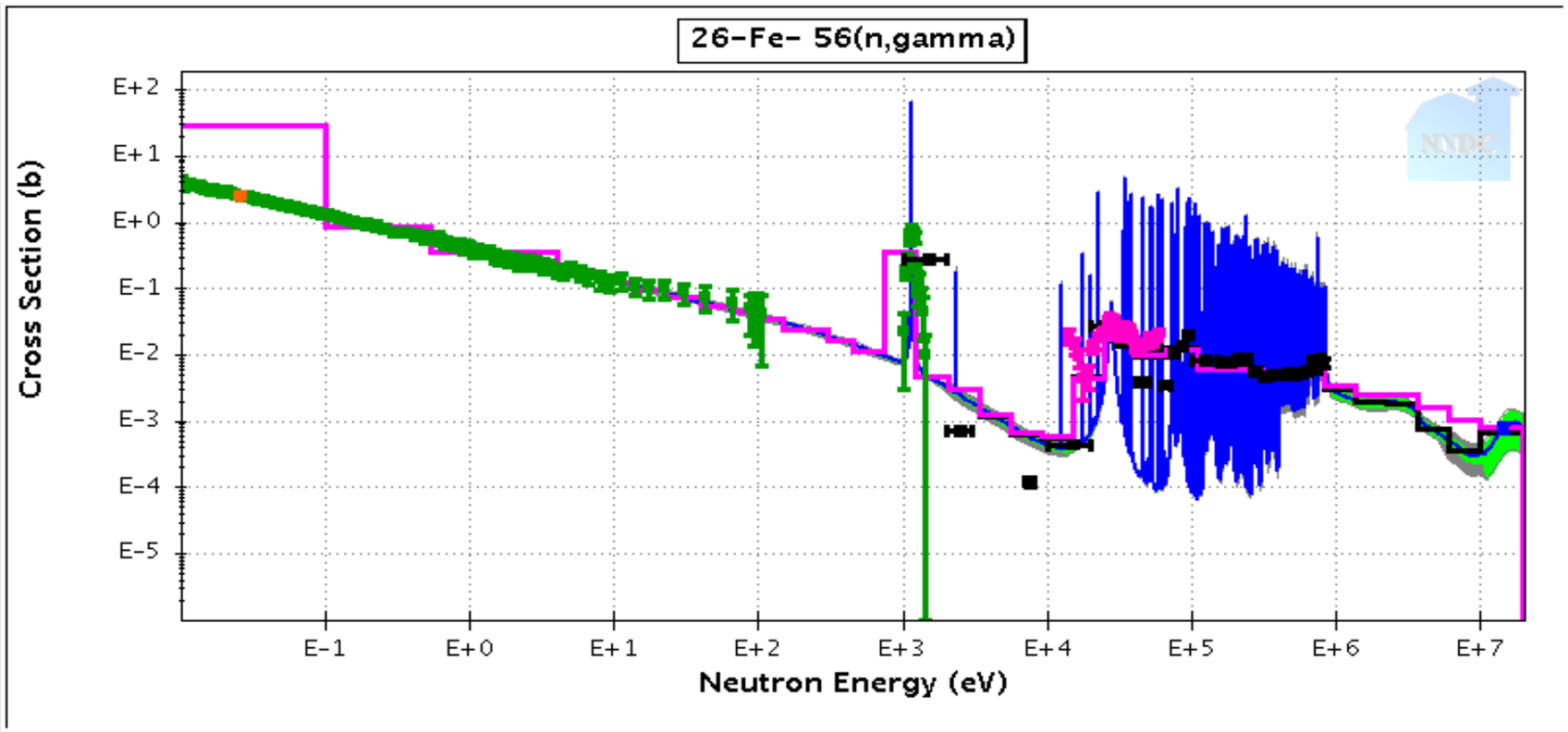
^{28}Si elastic integral quantities from Sigma-QA

Elastic						
Library	THERMAL	RI 0.5-2E+7 eV	MACS 30 keV	^{252}Cf	14 MeV	R' (fm)
ENDF/B-VII.0	1.992	3.882E+1	2.382	2.871	6.620E-1	4.136
JEFF3.1	1.992	3.885E+1	2.382	2.854	6.969E-1	4.136
JENDL4.0	1.992	3.904E+1	2.382	2.902	7.400E-1	4.136
ROSFOND	1.992	3.885E+1	2.382	2.854	6.969E-1	4.136
ENDF/B-VI.8	1.992	3.882E+1	2.382	2.871	6.620E-1	4.136
CENDL3.1	1.992	3.879E+1	2.382	2.884	6.424E-1	4.136
Atlas	1.992					4.800
Atlas Δ	6.000E-3 3.01E-1%					2.000E-1 4.16%
AFCI2.0 Δ	1.992E-2 1.00%	9.587E-1 2.46%	1.540E-1 6.46%	1.351E-1 4.70%	5.435E-2 8.20%	
Recommended Δ	6.000E-3 3.01E-1%					3.073E-1 7.43%

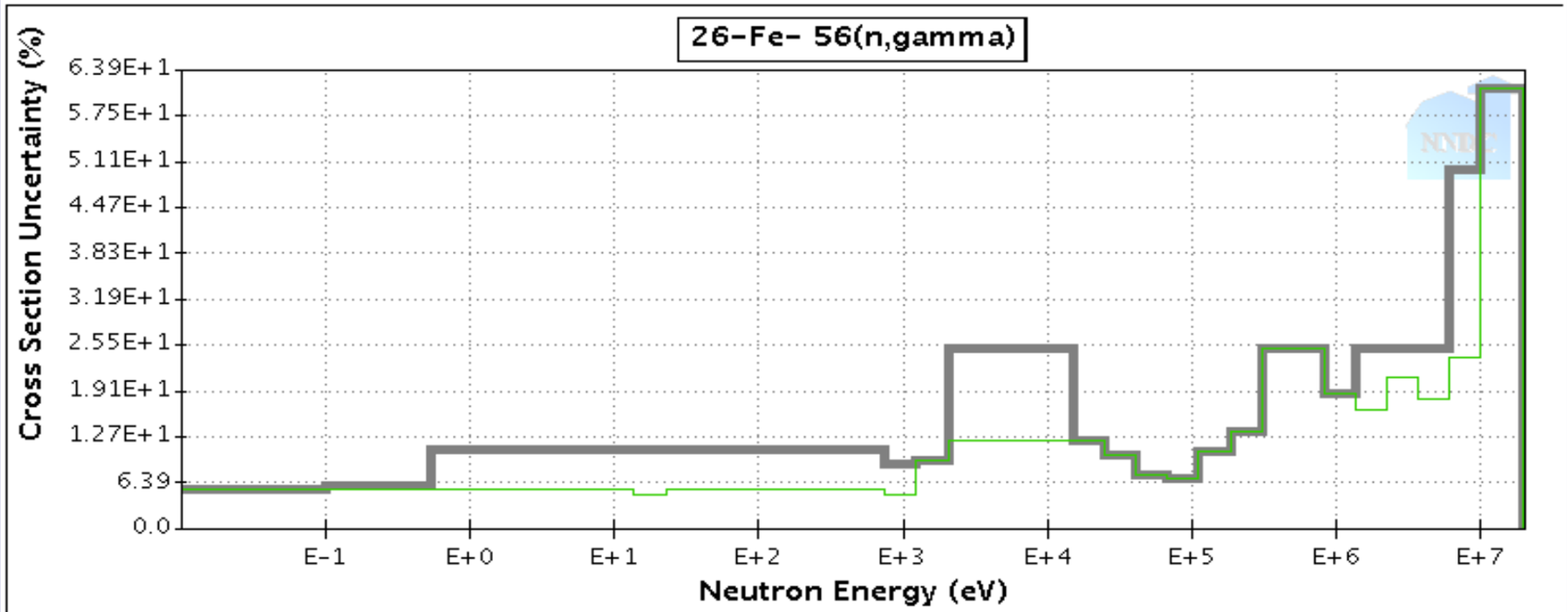
^{60}Ni capture integral quantities from Sigma-QA

Capture					
Library	THERMAL	RI 0.5-2E+7 eV	MACS 30 keV	^{252}Cf	14 MeV
ENDF/B-VII.0	2.772	1.412	2.826E-2	4.022E-3	2.859E-4
JEFF3.1	2.772	1.412	2.826E-2	6.033E-3	7.558E-4
JENDL4.0	2.913	1.472	2.792E-2	6.172E-3	6.992E-5
ROSFOND	2.772	1.412	2.826E-2	6.033E-3	7.558E-4
ENDF/B-VI.8	2.772	1.406	2.826E-2	4.022E-3	2.859E-4
CENDL3.1	2.772	1.413	2.826E-2	5.825E-3	1.131E-3
KADoNiS			2.990E-2		
Atlas	2.500	1.400			
Kadonis Δ			7.000E-4 2.34%		
Atlas Δ	6.000E-2 2.40%	2.000E-1 1.42E+1%			
AFCI2.0 Δ	1.386E-1 5.00%	1.183E-1 8.37%	1.811E-3 6.40%	2.902E-4 7.21%	6.591E-5 2.30E+1%
Recommended Δ	1.430E-1 5.15%	2.017E-1 1.42E+1%	7.968E-4 2.81%		

Example of Sigma-QA plot



Cursor at: x = 3.6826E-2 (eV) y = 2.0212E-2 (b)



Update Plot | Reset

1E-2 ≤ E_n (eV) ≤ 2E7 Log

1E-6 ≤ σ (b) ≤ 1.924E2 Log

- [x] ENDF/B-VII.0 pointwise
- [ ] AFCI 1.2 uncertainty
- [x] AFCI 1.3 uncertainty
- [ ] AFCI 2.0 uncertainty
- [x] AFCI 2.0' uncertainty

Group cross sections with 1/E flux

- [x] ENDF/B-VII.0 group
- [x] JENDL-4.0 group
- [ ] JEFF-3.1 group
- [ ] CENDL-3.1 group
- [ ] ROSFOND group
- [ ] ENDF/B-VI.8 group

There are 7 EXFOR datasets

- [ ] Check/Uncheck All
- [x] Huang Zheng-De 1980
- [x] Shcherbakov 1977
- [x] Shcherbakov 1977
- [x] Allen 1982
- [ ] Allen 1976
- [x] Macklin 1964
- [x] Pomerance 1952

Remove EXFOR

Download plot for your article

Quality Assurance (continued)

- Code ‘unCor’, (Mattoon, Oblozinsky) checks the library for possible problems in uncertainties and/or correlations

```
Uncertainties too large: 19 total

MT16  in  001_H_002, max = 100.00%
MT102 in 003_Li_007, max = 100.00%
MT4   in  005_B_010, max = 100.00%
MT102 in 040_Zr_090, max = 100.00%
MT102 in 040_Zr_095, max = 100.00%
MT2   in 040_Zr_095, max = 100.00%
MT51  in 090_Th_232, max = 100.00%
MT852 in 090_Th_232, max = 100.00%
MT18  in  092_U_238, max = 100.00%
MT102 in 094_Pu_238, max = 100.00%
MT4   in 094_Pu_238, max = 100.00%
MT102 in 094_Pu_240, max = 100.00%
MT102 in 094_Pu_241, max = 100.00%
MT102 in 094_Pu_242, max = 100.00%
MT102 in 095_Am_242m, max = 100.00%
MT102 in 096_Cm_242, max = 100.00%
MT18  in 096_Cm_242, max = 100.00%
MT4   in 096_Cm_242, max = 100.00%
MT102 in 096_Cm_244, max = 100.00%

Uncertainties too small: 55 total

MT1   in  001_H_001, min = 0.29% in bin 33 (27 bins < 1%)
MT2   in  001_H_001, min = 0.30% in bin 12 (27 bins < 1%)
MT1   in 002_He_004, min = 0.50% in bin 11 (28 bins < 1%)
MT2   in 002_He_004, min = 0.50% in bin 11 (28 bins < 1%)
MT1   in 003_Li_006, min = 0.20% in bin 30 (21 bins < 1%)
MT105 in 003_Li_006, min = 0.20% in bin 18 (25 bins < 1%)
MT1   in 003_Li_007, min = 0.27% in bin  3 (7 bins < 1%)
MT2   in 003_Li_007, min = 0.42% in bin  4 (6 bins < 1%)
MT1   in 004_Be_009, min = 0.50% in bin 24 (14 bins < 1%)
MT2   in 004_Be_009, min = 0.50% in bin 24 (14 bins < 1%)
```

Uncor results

- Test for small uncertainties:

Total with small uncertainties : 173

Reaction	Max %unc
(n,total)	1.0
(n.elas)	2.0
(n.inel)	3.0
(n,2n)	3.0
(n,f)	0.7
(n,g)	2.0
Total nubar	0.7
Prompt nubar	0.7
other	2.0

Uncor results (cont.)

- 2nd test for too small uncertainties:

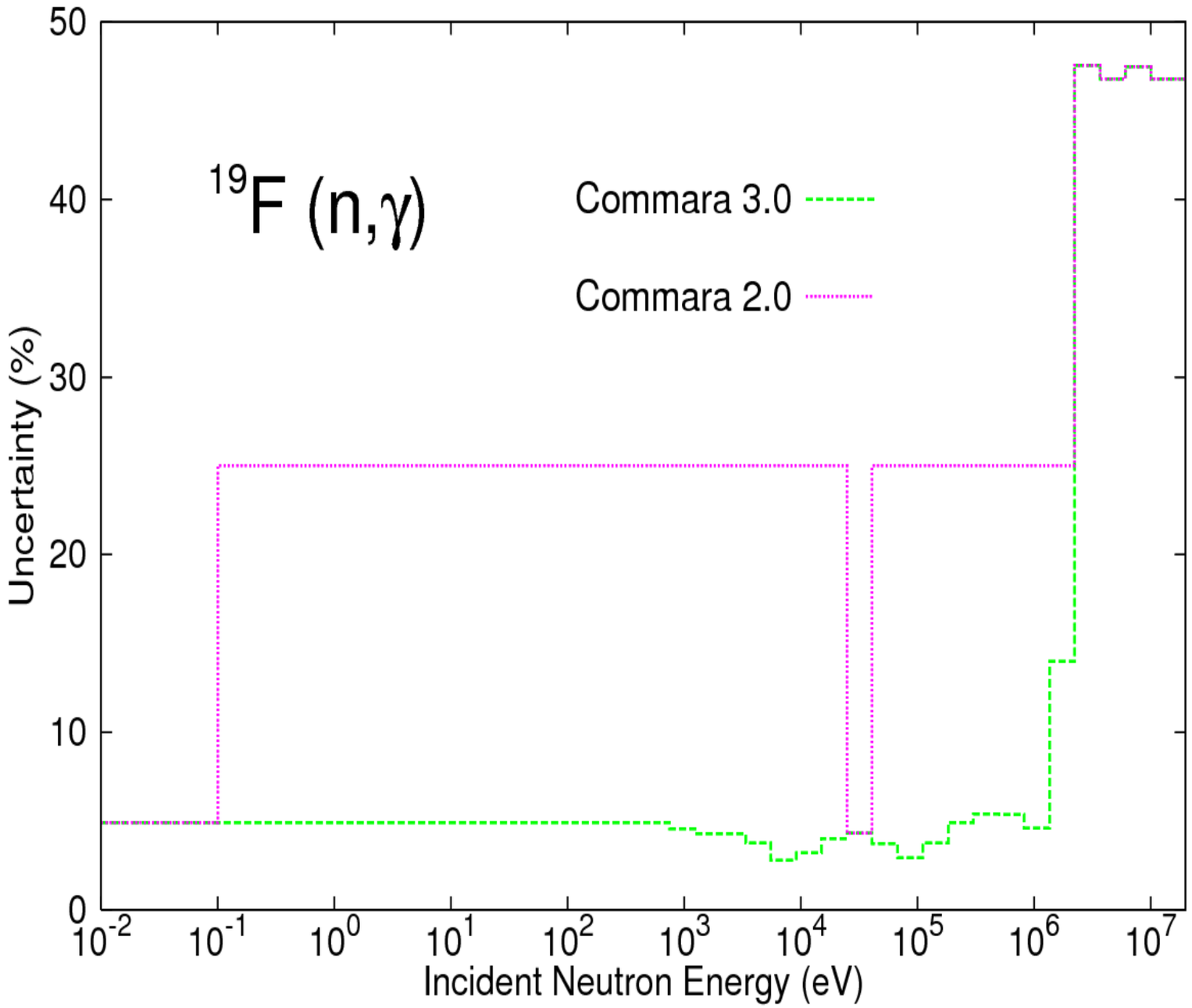
  If cross section < 3mb, min uncert = 25%

  Total materials : 127

- Optical model peaks in elastic: 44
- Zero unc with non-zero cross section: 98
- Peaks/jumps in uncertainties: 55
- Negative eigenvalues : only PFNS (small)

^{19}F

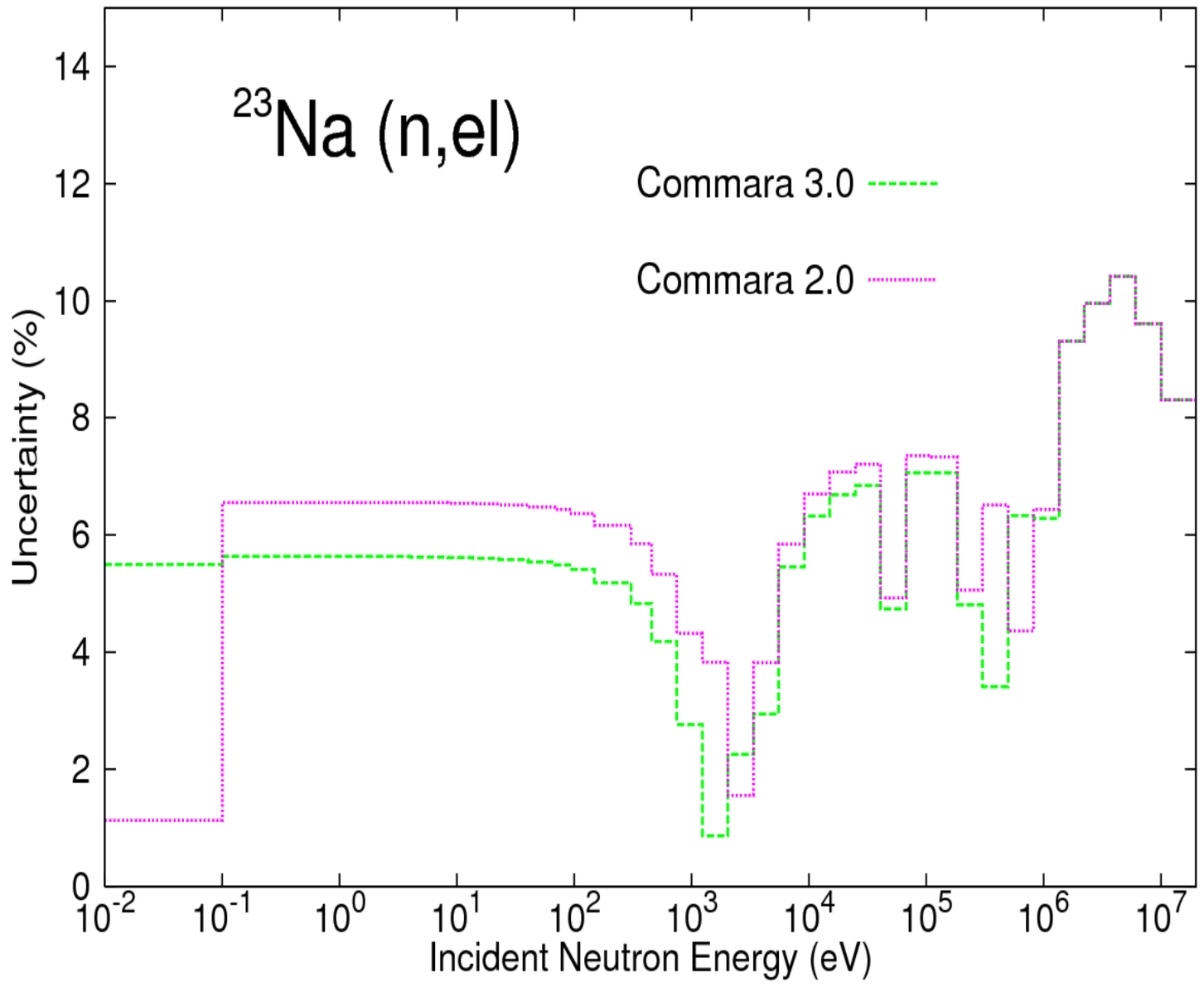
- New ORNL evaluation



^{23}Na

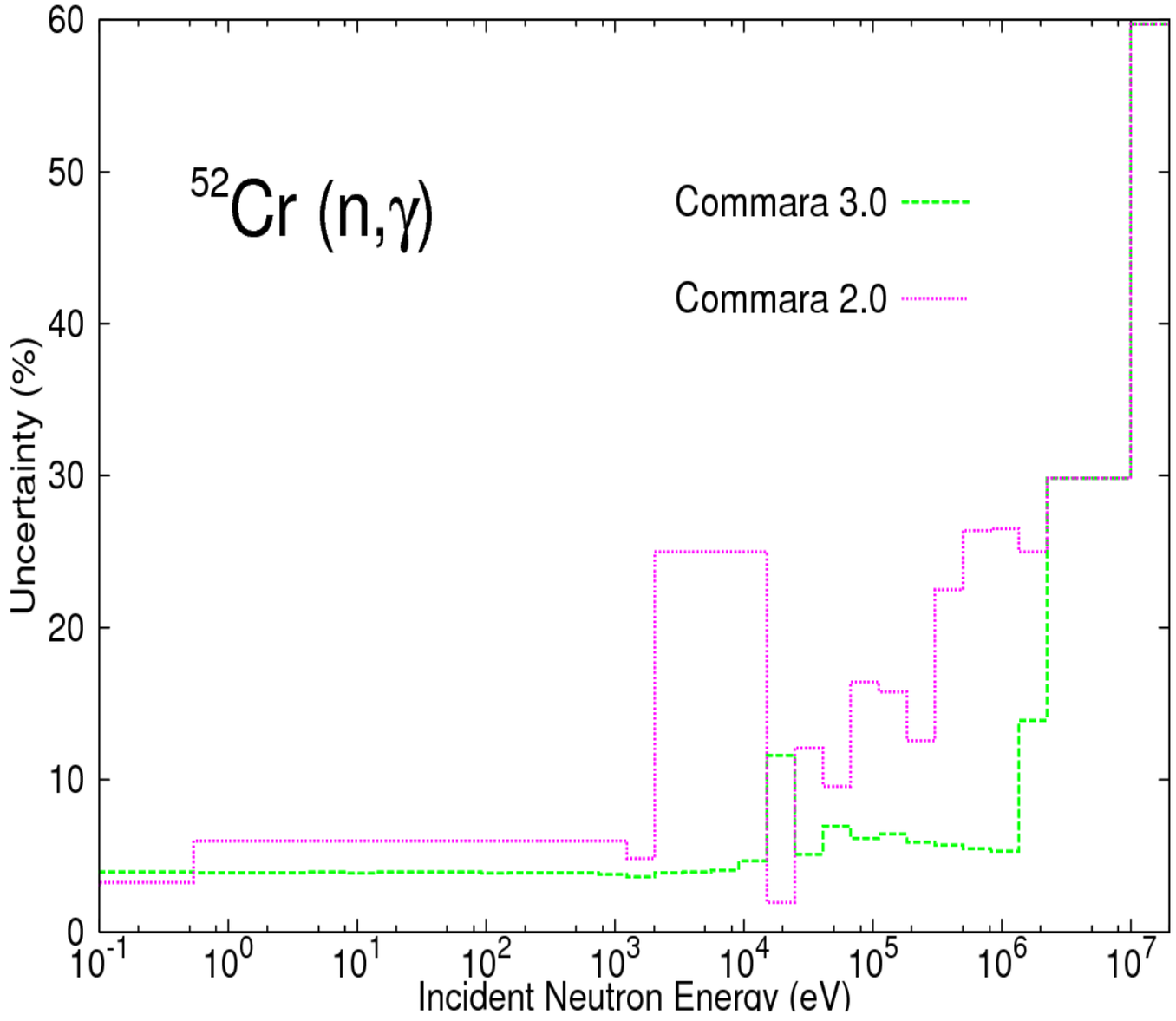
Reverted to ENDF/B-VII.0

COMMARA 2.0 Covariances backported, w/ modifications



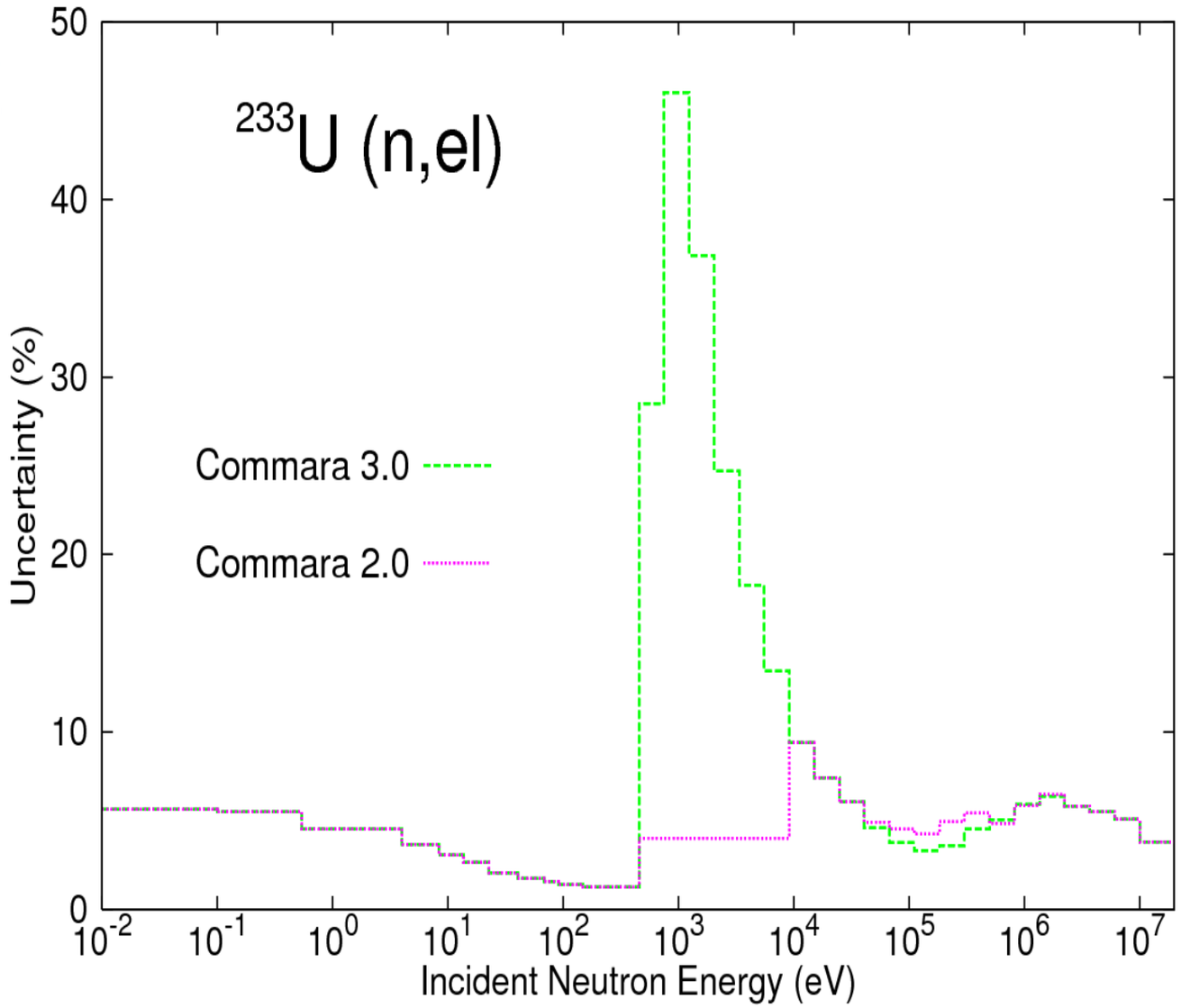
^{52}Cr

- New ORNL RRR evaluation



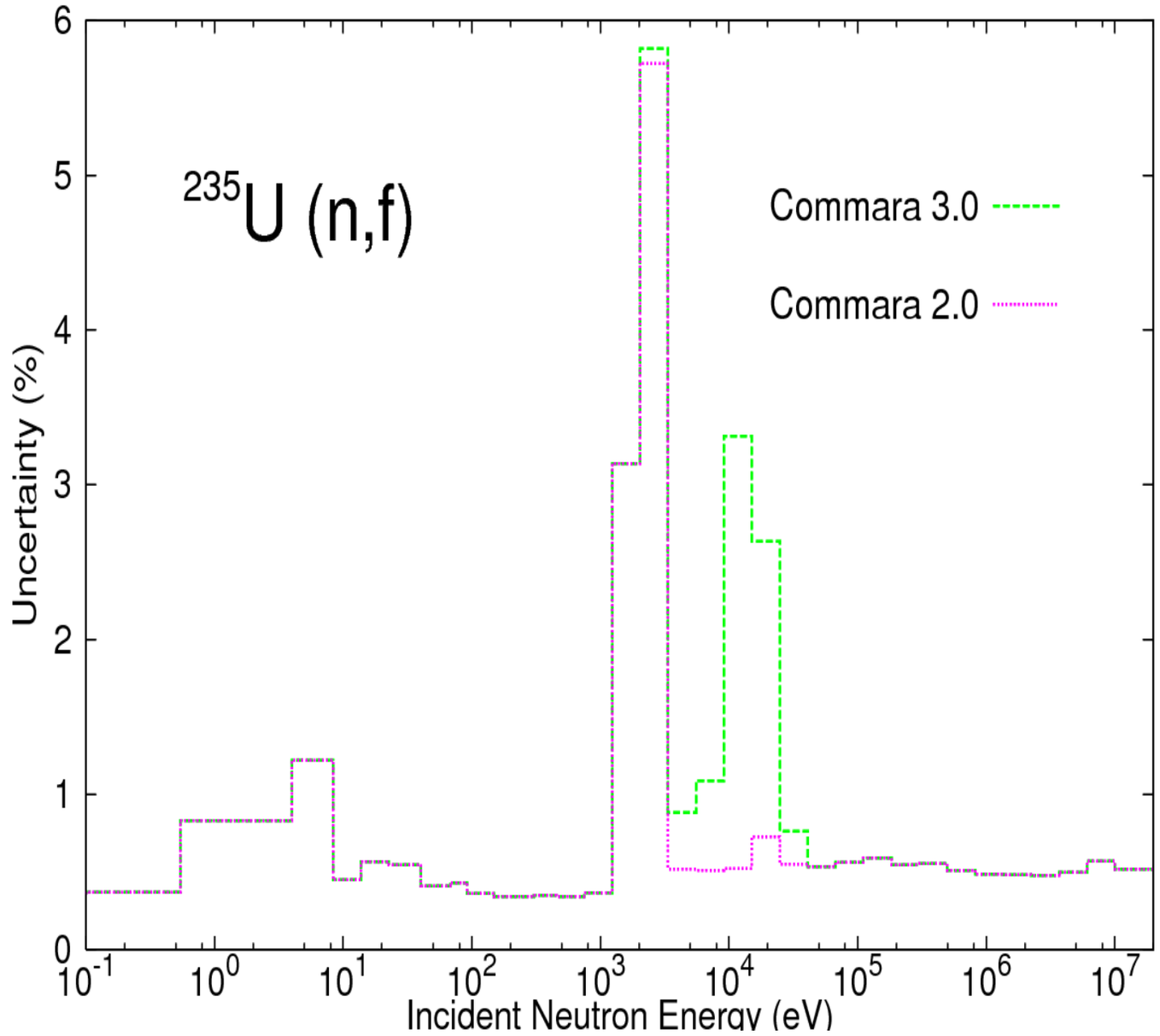
^{233}U

- ORNL RRR MF32 replaced with LANL MF33



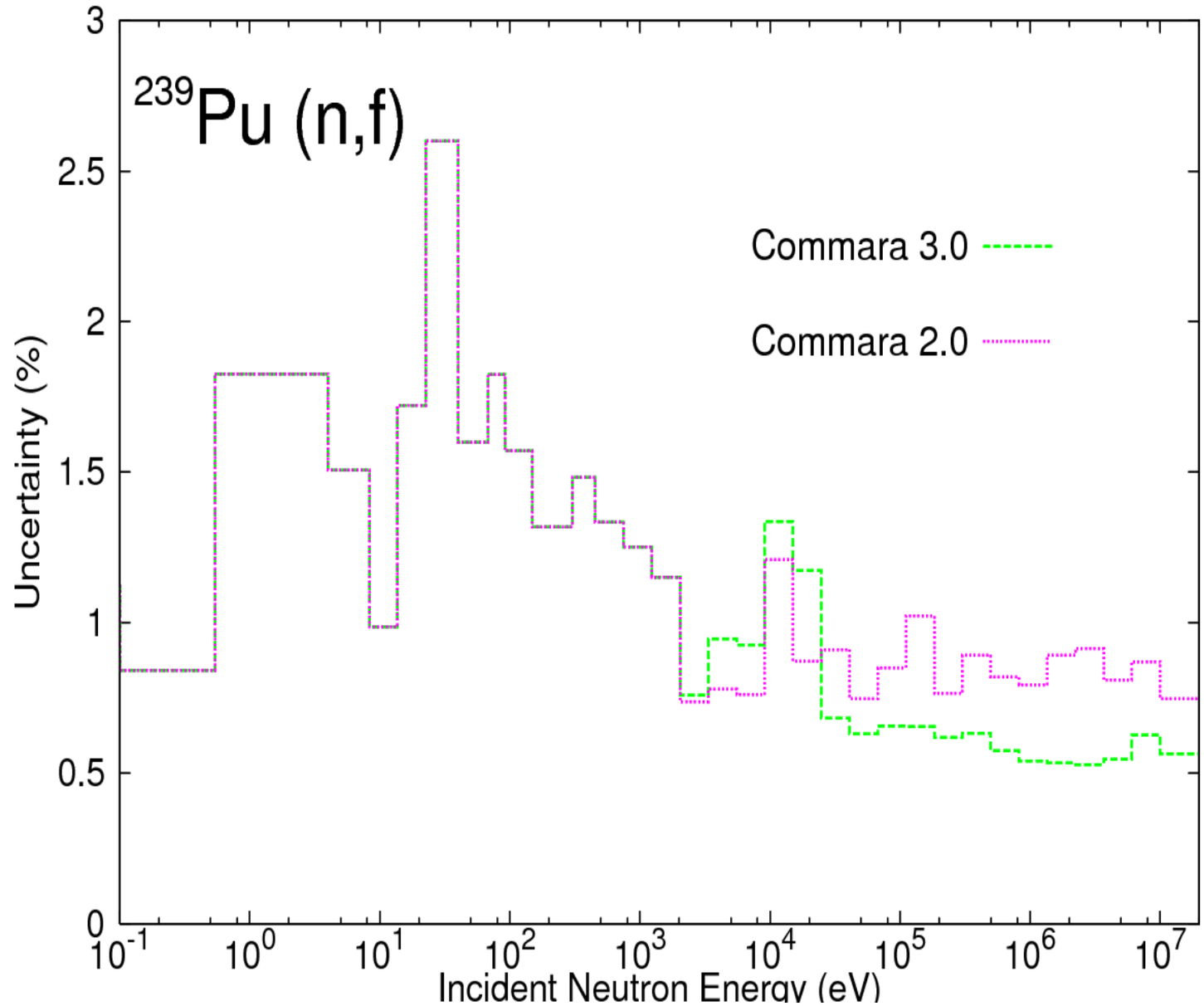
^{235}U

- New MF33 eval by LANL/ORNL



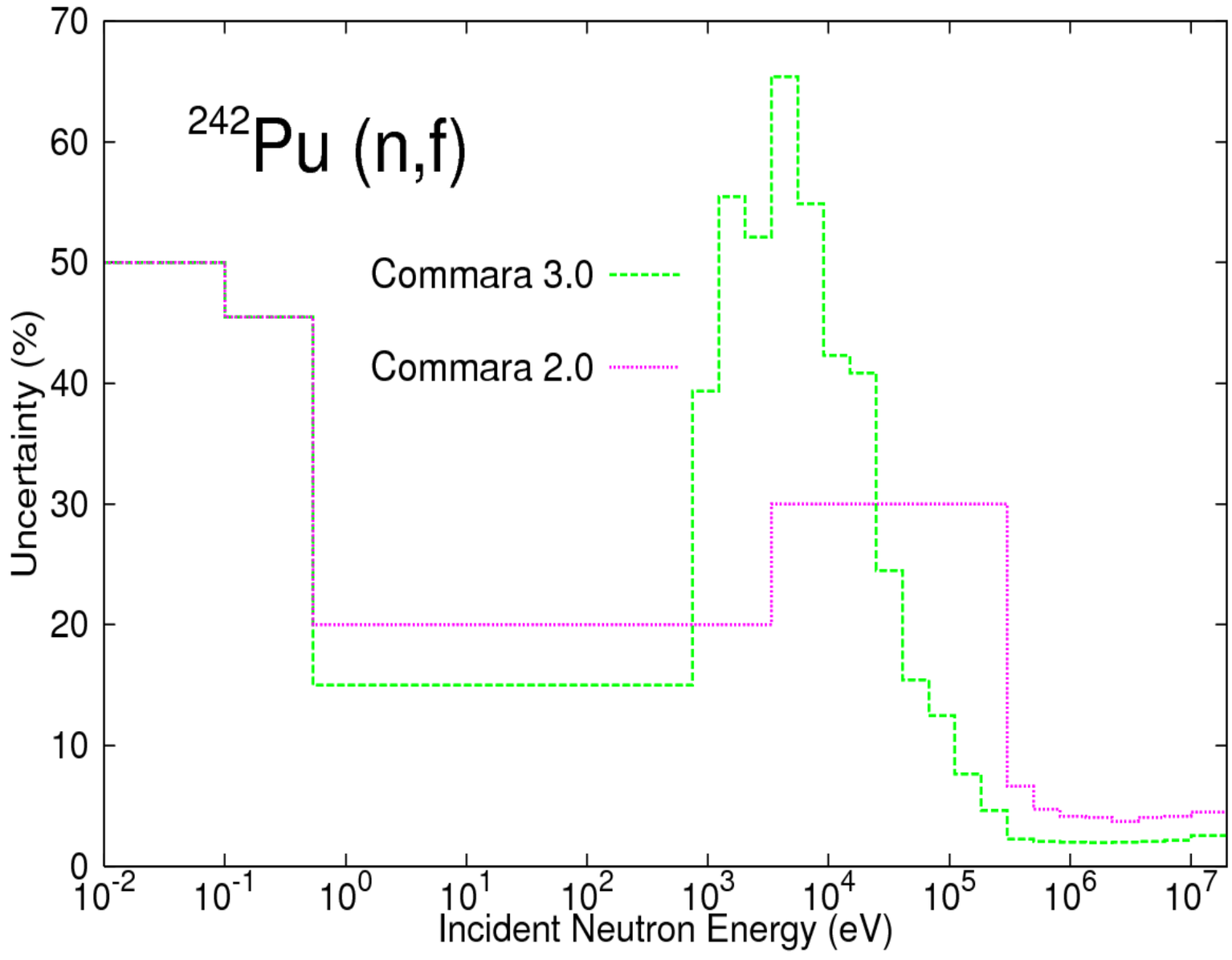
^{239}Pu

- New MF33 eval by LANL/ORNL



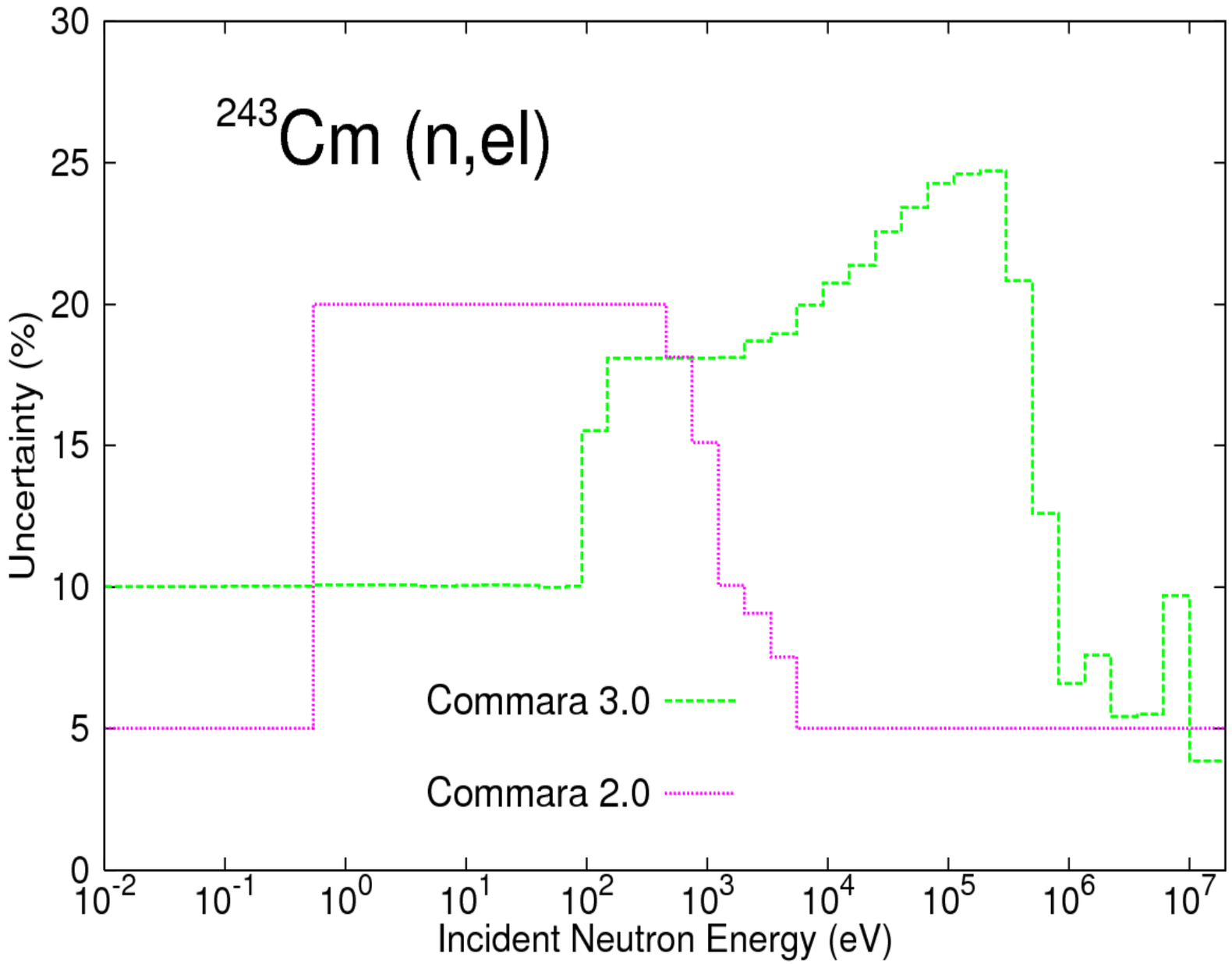
^{242}Pu

from JENDL-4



^{243}Cm

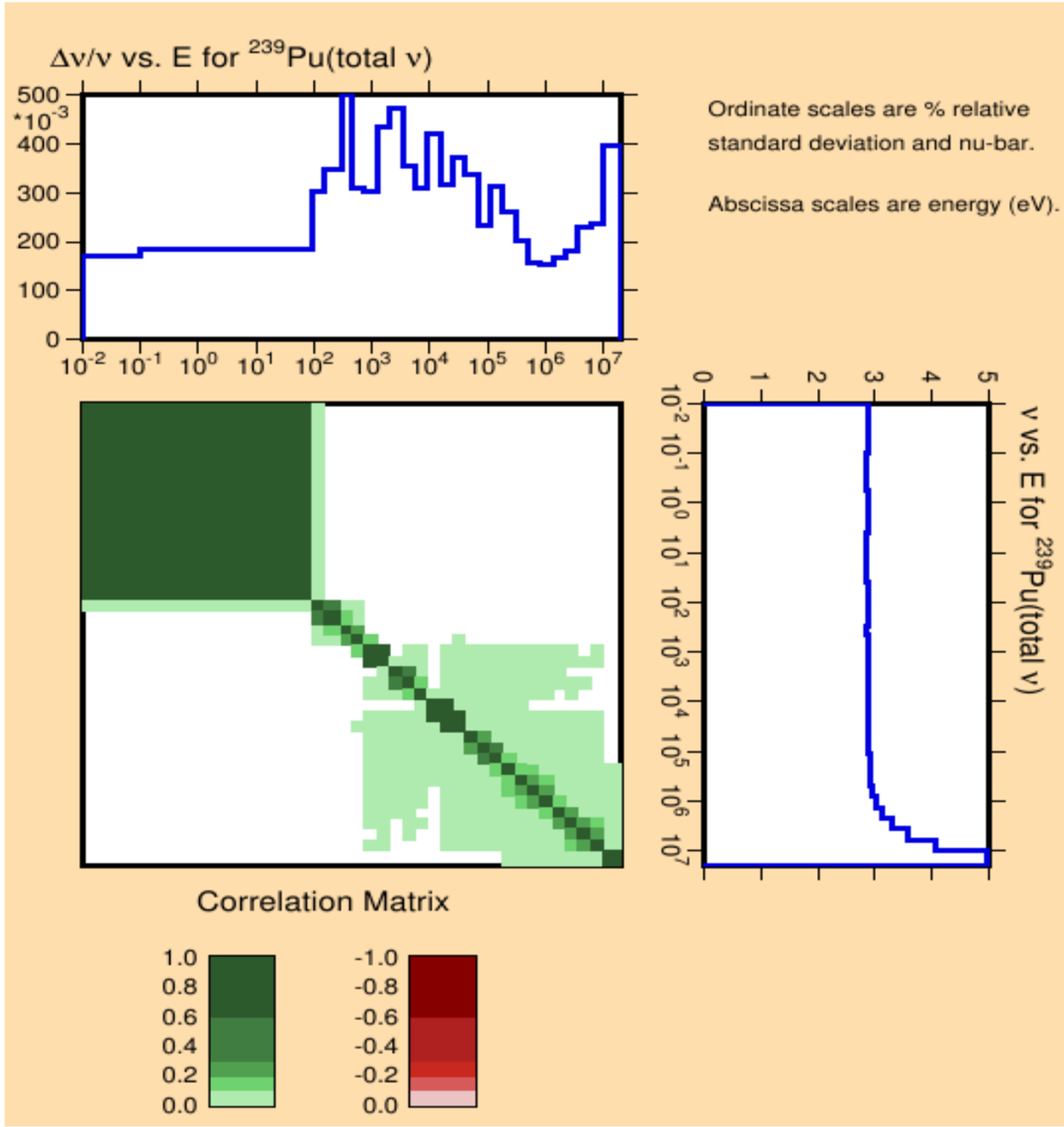
from JENDL-4



Nubar covariances

- ^{239}Pu

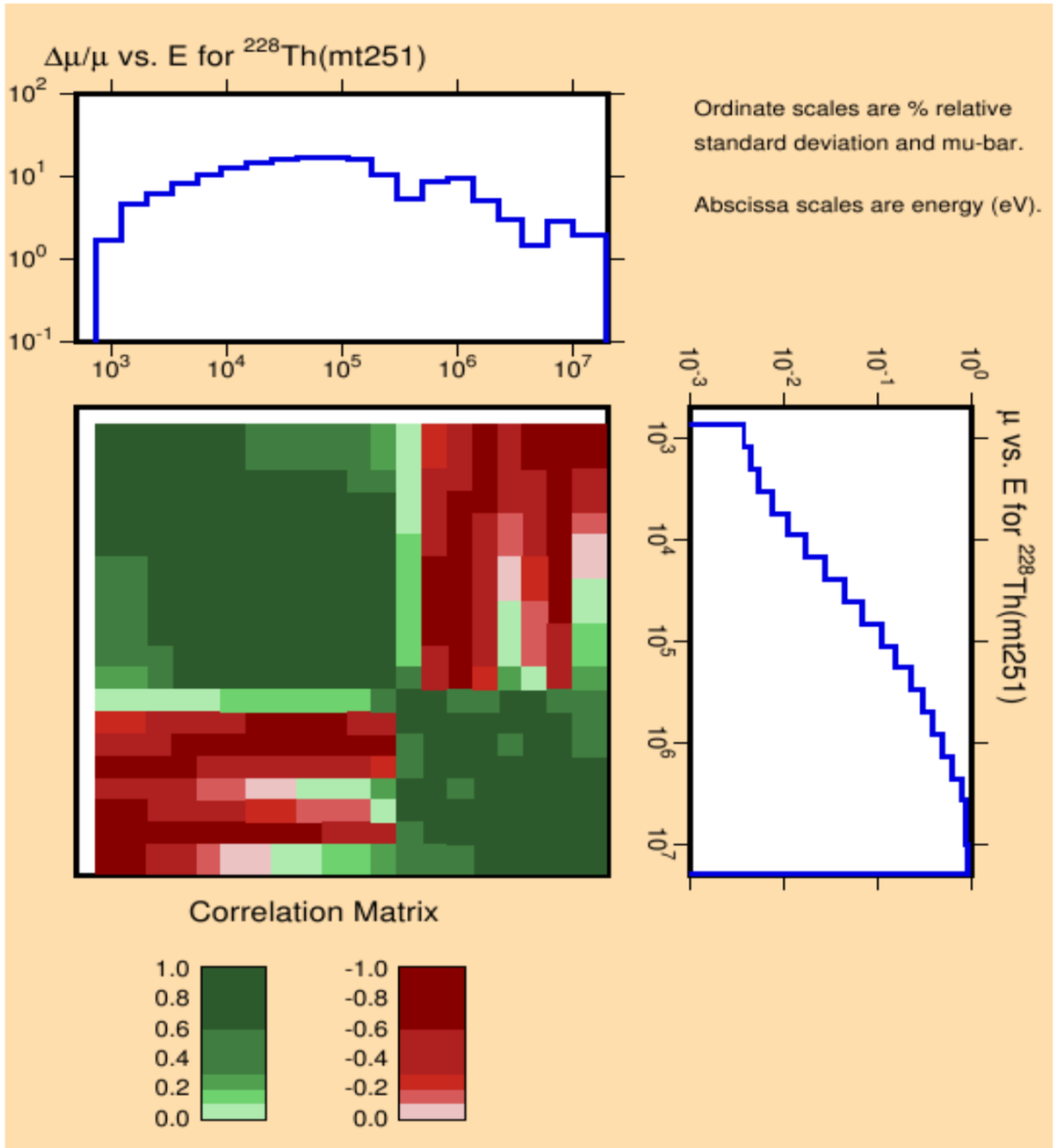
From LANL



Mubar covariances

^{228}Th

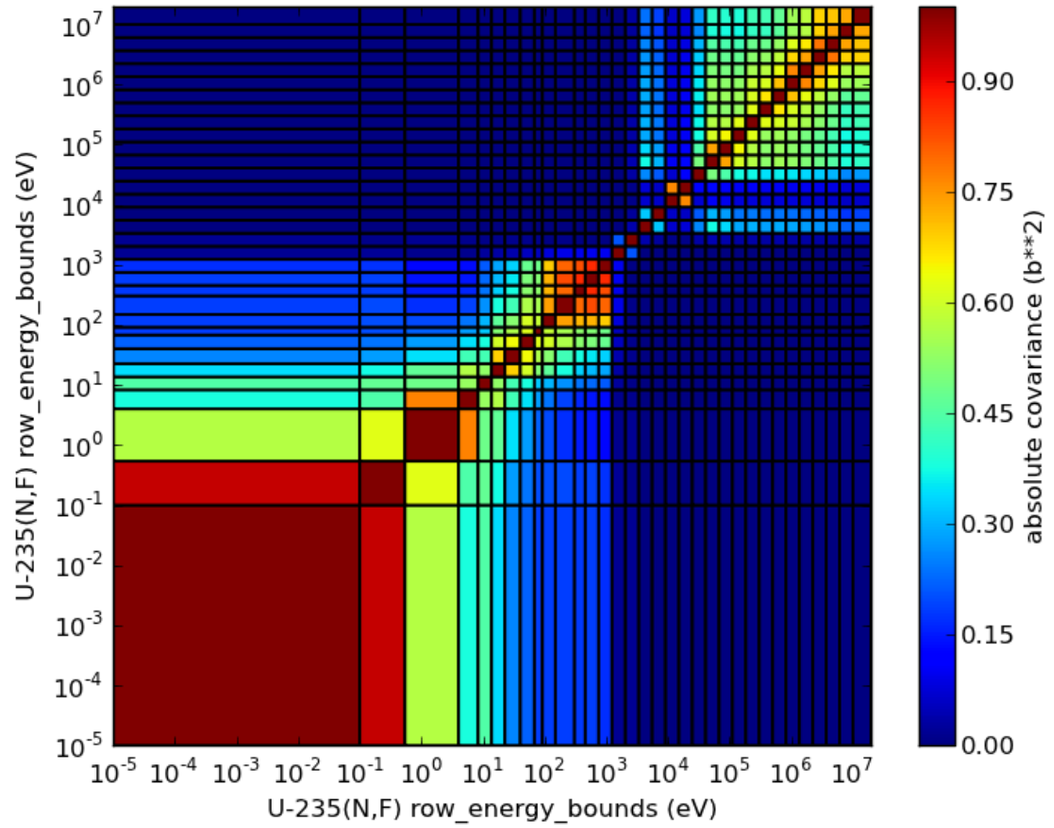
JENDL-4

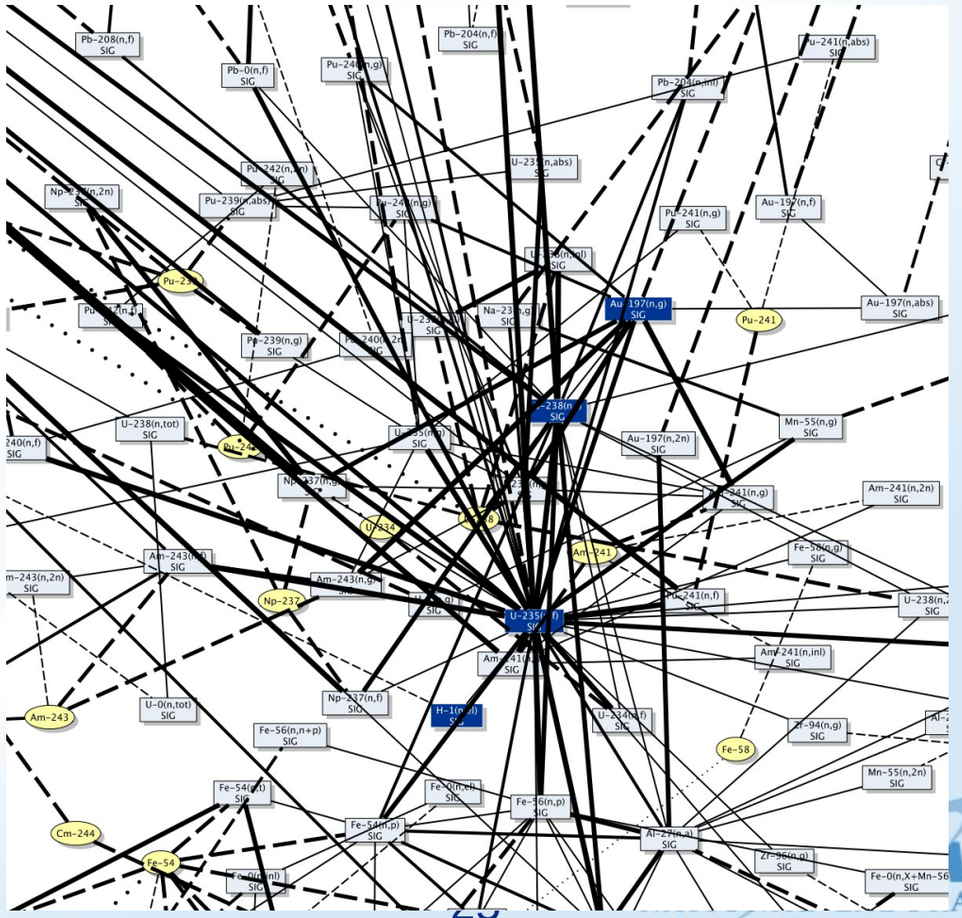


Developing cross-isotope cross-reaction covariances

- EXFOR contains many forms of coupled cross section data:
  - reaction combinations
  - reactions on elemental targets
  - non-elastic data
  - ratios to monitors (unused)
  - “isomers math” (unused)
- Goal: Refit all COMMARA-2.0 priority cross sections simultaneously
  - 33-group structure
  - ENDF/B-VII.1 covariances for prior
    - includes “on-diagonal” covariance
    - includes standards “off-diagonals”
  - use EXFOR entry’s REACTION string to construct linearized kernel

End product is covariance, not mean values





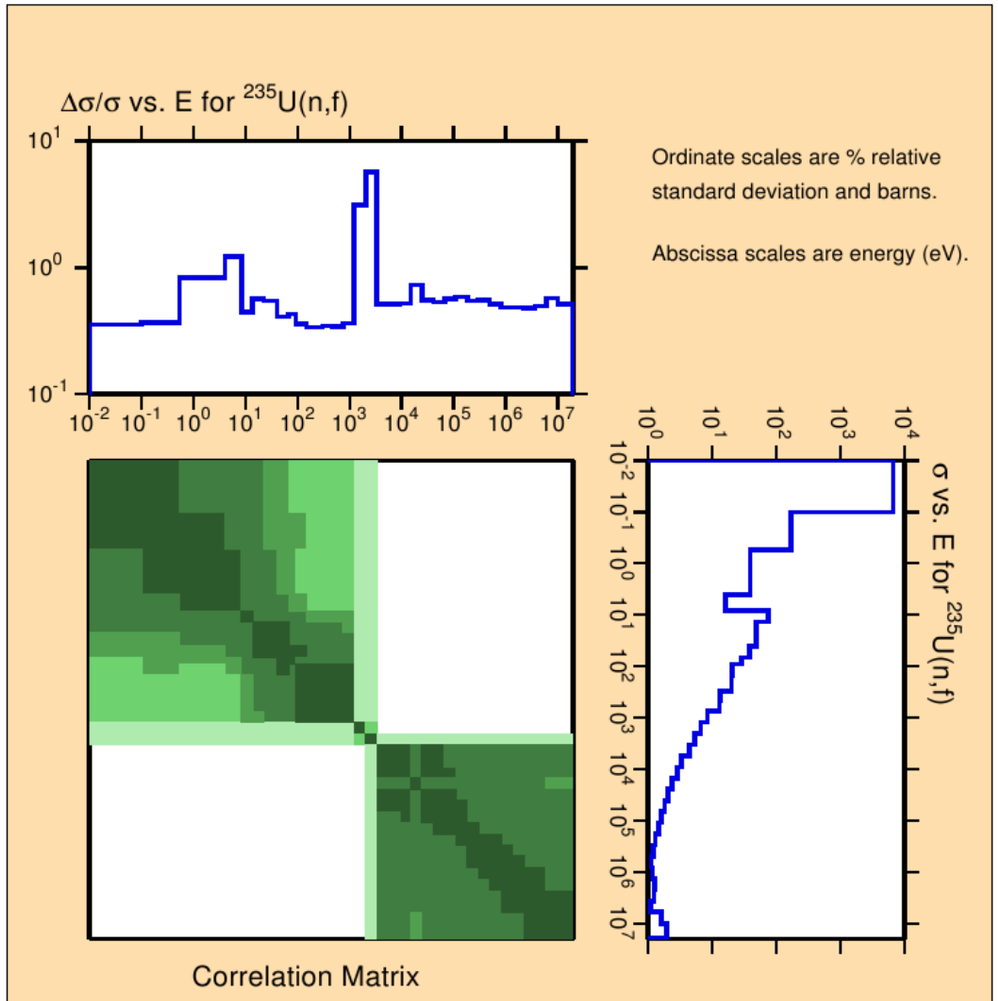
Summary

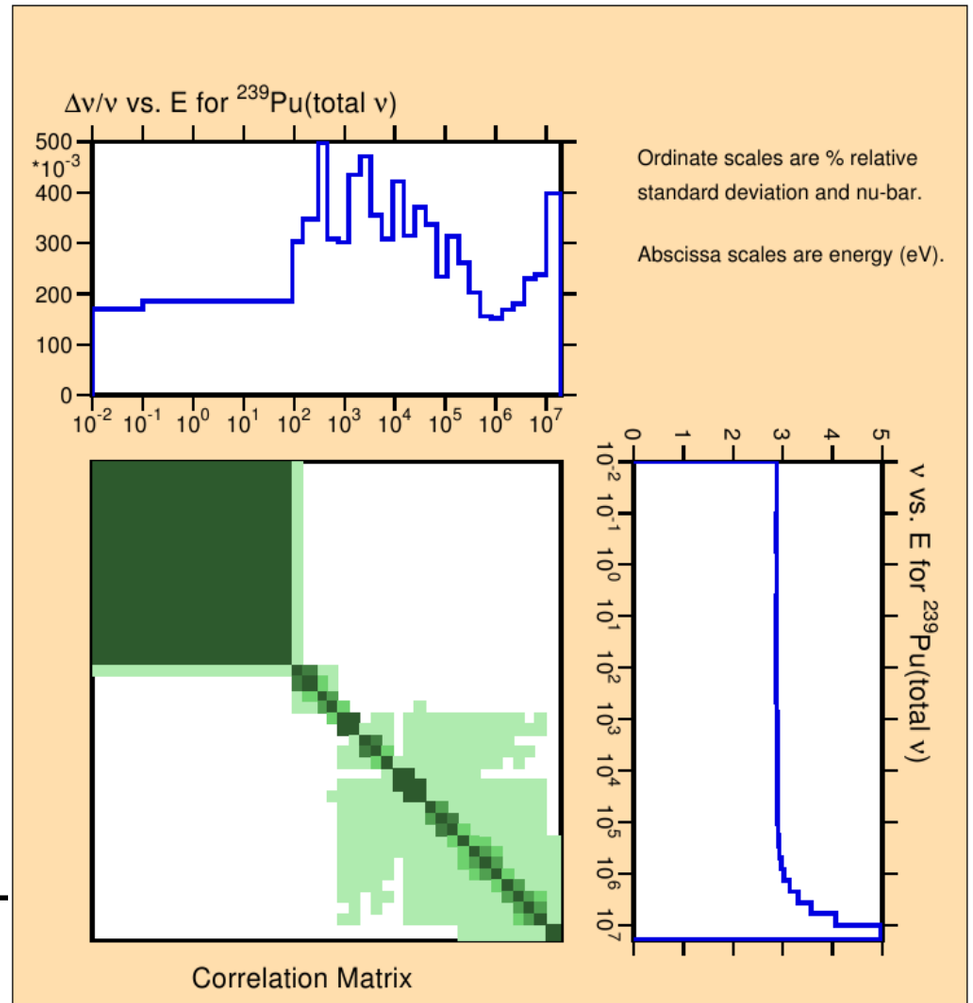
- ENDF/B-VII.1 released Dec 2011
- Files tested using Sigma-QA and UnCor procedures for Quality Assurance
- ENDF files processed with NJOY using AFCI 33-group structure, flux = thermal, 1/E, fiss spec (E>100 keV)
- Subset used for COMMARA-3.0 in SG33 format

EXTRAS

33-group structure

- AFCI is a processed covariance library, using 33 energy bins.
- Group boundaries chosen so that in most cases, $\ln(E_2/E_1)=0.5$





Which materials to include/modify

- Include all materials with covariance data from ENDF/B-VII.1?
- Only replace materials in COMMARA-2.0?
- Limit max uncertainty to 100%?
- Place any limits on minimum uncertainties?
- Same reaction channels ➠ SG33?

Contents of Library

- Major Actinides produced by LANL/ORNL
  - ^{233,235,238}U, ^{239}Pu – simultaneous evaluation
- Structural materials produced by BNL
  - ^{23}Na, ^{52}Cr, ^{56}Fe, ^{58}Ni, Pb, Bi, ...
- Light nuclei from LANL/ORNL (R-matrix, also low-fi)
- Minor Actinides partly based on V. Maslov estimates, partly on BNL and LANL efforts, JENDL
- Fission products most based on low-fidelity covariances
- Mubar covariances for ^{23}Na, ^{56}Fe
- Prompt fission neutron spectra for ^{238,239,240}Pu

Integral Weightings