²³⁹Pu Resonance Region Evaluation



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Cross Section Evaluation Working Group Brookhaven National Laboratory November 15-17, 2011



Motivations for a New ²³⁹Pu Evaluation

- Existing resonance parameter (RP) representation done with three disjoint resonance parameter sets as 1.0×10⁻⁵ eV to 1 keV, 1 keV to 2 keV, 2 keV to 2.5 keV;
 - ✓ Cross section mismatch at the energy boundaries;
 - ✓ Not easy to generate uncertainty for the whole energy region (zero correlation);
 - New evaluation: single resonance parameter set covering the energy range 1.0×10^{-5} eV to 2.5 keV

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- Resonance parameter covariance generated
- Solve a long standing problem for thermal benchmark prediction;

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Experimental Data Sets Used in the RR Evaluation

Reference	Energy Range	Facility	Measurement
	(eV)		
Bollinger et al. (1956)	0.01 - 1.0		Total Cross Section
Gwin et al. (1971)	0.01 - 0.5	ORELA	Fission and Absorption at 25.6 m
Gwin et al. (1976)	1.0 - 100.0	ORELA	Fission and Absorption at 40.0 m
Gwin et al. (1984)	0.01 - 20.0	ORELA	Fission at 8 m
Weston et al. (1984)	9.0 - 2500.0	ORELA	Fission at 18.9 m
Weston et al. (1988)	100.0 - 2500.0	ORELA	Fission at 86 m
Weston et al. (1993)	0.02 - 40.0	ORELA	Fission at 18.9 m
Wagemans et al. (1988)	0.002 - 20.0	GELINA	Fission at 8 m
Wagemans et al. (1993)	0.01 - 1000.0	GELINA	Fission at 8 m
Harvey et al. (1985)	0.7 - 30.0	ORELA	Transmission at 18 m
Harvey et al. (1985)	30.0 - 2500.0	ORELA	Transmission at 80 m



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Transmission

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Effective System Multiplication Factor: k_{eff}

Production





6 Managed by UT-Battelle for the Department of Energy Multiplication Factor: k_{∞}

Leakage

 \mathcal{VO}_{f}

a

 $k_{\infty} \propto \eta =$

Production

Absorption

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Issues with ORNL Evaluation

- Results of plutonium solution calculations indicate no improvement using ORNL evaluation. Longstanding problem persists!!
- Review of the ²³⁹Pu is underway
- Parts involved are:

ORNL, LANL, CEA and others!!



International Community Effort:

- Working Party on International Evaluation Co-operation (WPEC) subgroup created
- ✓ Objective: Address issues on the discrepancies of Pu-SOL-THERMAL assemblies and Pu-INTER assemblies calculations
- ✓ Strategy
 - Use New Leal/Derrien ENDF resonance evaluation and covariance
 - Use sensitivity analysis tools to indentify which parameters are important on both differential and integral data adjustment
- ✓ Goal: obtain a 239 Pu resonance evaluation that :
 - Represent the differential data well,
 - leads to improvements in calculations of integral data



Effective Work

- Choice of benchmark problems :
 - Define a set of benchmarks sensitive to ²³⁹Pu nuclear data from ICSBEP and IRPhEP.
 - Common Benchmarks : ICSBEP ²³⁹Pu benchmark systems Water-Reflected and bare spheres of plutonium nitrate solutions

Intermediate and fast Benchmarks will be added

ORNL/CEA

 Perform calculations of these benchmarks with various evaluations (ENDF, JEFF, JENDL) using Monte-Carlo and Deterministic codes

Skip Kahler of LANL indentified a subset of 15 Pu-Sol-Therm benchmarks in the ICSBEP that can be used to address the problem.



²³⁹Pu Data Sensitivity and Adjustment at ORNL

✓Use²³⁹Pu resonance evaluation with covariance done at ORNL

✓ Process the evaluation with the AMPX/PUFF code system to generate group cross sections and covariance

✓44-neutron group structure of the SCALE system was used

- ✓ 15 ICSBEP ²³⁹Pu benchmark calculations
 - Thermal water reflected benchmark experiments were used
- \checkmark Sensitivity calculations were done with the TSUNAMI code
- ✓ Data adjustments were done with the TSURFER code



TSUNAMI Analysis for Cross-Section Evaluations

- TSUNAMI S/U capability invaluable tool for cross-section evaluation
 - Provides improved understanding of nuclear data physics for specific applications
 - Identify parameters and energy regions of importance

$$S = \frac{\sigma_{\chi}}{k} \frac{\partial k}{\partial \sigma_{\chi}}$$
 and $V = S C S^{t}$

- TSUNAMI used in support of the NCSP and DOE/RW fission program
 - Nuclear Data evaluator performs sensitivity analysis of critical experiment to understand the physics of the problem and identify energy regions that are "exercised" by the criticals



Consolidation of Computed and Measured Responses Using <u>Generalized Linear Least-Squares (GLLS)</u>

- GLLS consolidates calculations with measured responses
- Computes "best" data adjustments to eliminate differences
- Results in more consistent results with lower uncertainties
- Propagation of data adjustments to a proposed design system provides computational bias and uncertainty



Application of GLLS to Data Adjustment

M-dimensional discrepancy vector:

$$d(\alpha, K_m) = K_c(\alpha) - K_m$$

computed measured

GLLS determines modified nuclear data α ' and measured responses K'_m such that . . .

- Discrepancy vector $d(\alpha', K'_m) \rightarrow 0$
- Uncertainties/correlations in α and K_m (i.e., $C_{\alpha\alpha}$ and C_{mm} respectively) are taken account
- Overall consistency maximized by minimizing chi-squared:

$$\chi^2 = [\boldsymbol{\alpha'} - \boldsymbol{\alpha}]^{\mathsf{T}} [\boldsymbol{C}_{\boldsymbol{\alpha}\boldsymbol{\alpha}}]^{-1} [\boldsymbol{\alpha'} - \boldsymbol{\alpha}] + [\boldsymbol{K'}_m - \boldsymbol{K}_m]^{\mathsf{T}} [\boldsymbol{C}_{\mathsf{mm}}]^{-1} [\boldsymbol{K'}_m - \boldsymbol{K}_m]$$

overall adjustments to data, in units of variance

overall adjustments to measurements, in units of variance





Covariance Work ²³⁹Pu ORNL fission/capture estimation





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Benchmarking/Integral Data feed-back ²³⁹Pu Data Sensitivity and Adjustment at ORNL

Pu-Sol-Therm-021 Case 7



ORNL ²³⁹Pu sensitivity calculations of the cross section to k_{eff} (TSUNAM)

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Benchmarking/Integral Data feed-back ²³⁹Pu Data Sensitivity and Adjustment at ORNL

4.5 pu-239 fission pu-239 n,gamma 4.0pu-239 nubar pu-239 chi 3.5 Relative Change in Cross-section (%) 3.0 2.5 2.0 1.5 1.00.5 0.0-0.5 -1.0-1.5 -2.0 -2.5 -3.01.0E-04 1.0E-03 1.0E-02 1.0E-01 1.0E00 1.0E01 1.0E02 1.0E03 1.0E04 1.0E05 1.0E06 1.0E07 Energy (eV)

ORNL New Resonance + Covariance

ORNL²³⁹Pu data adjustment for the fifteen benchmark experiments (TSURFER)

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ORNL and CEA/Cadarache Work

ORNL/CEA

- Use of sensitivity analysis (combine Microscopic and Integral experiments) to help improvement of nuclear data
 - KENO/TSUNAMI/TSURFER Code at ORNL
 - ERANOS/SNATCH/CONRAD Code at CEA

ORNL/CEA

- Calculate effects of using Maslov PNFS



CEA work on the effect of ²³⁹Pu PNFS on Benchmarks

Solutions performed with TRIPOLI-4 Release 4.6

SMaslov ²³⁹Pu prompt fission neutron spectra replacement in ²³⁹Pu JEFF-3.1.1 evaluation file

• Personal communication June 2009

SICSBEP PU-SOL-THERM 001 and Pu-MET-FAST benchmarks

- MCNP data file automatic conversion
- TRIPOLI-4 MCNP4C3 k_{eff} calculations checks



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CEA work on the effect of ²³⁹Pu PNFS on Benchmarks

♥PU-SOL-THERM 001

• Water reflected 11.5 inch diameter spheres of plutonium nitrate solutions

	TRIPOLI4	b	TRIPOLI4	υ	Discrpancy	σ
	Result		Result		MASLOV	
	CEA2005		CEA2005			
			MASLOV			
1.T8A	1,00252	47	1,00587	45	335	65
2.T8A	1,00374	48	1,00755	48	381	68
3.T8A	1,00631	46	1,01066	48	435	66
4.T8A	1,00137	47	1,00575	46	438	66
5.T8A	1,00446	48	1,00923	49	477	69
6.T8A	1,00779	50	1,01061	48	282	69
				MIN	282	
				MOY	391	
				MAX	477	

CEA work on the effect of ²³⁹Pu PNFS on Benchmarks

₿PU-MET-FAST

Bare spheres of Pu (001, 002)

Reflected spheres of Pu (005 W, 008 Th, 009 Al, 010 U, 011 Water, 018 Be)

		TRIPOLI4	σ	TRIPOLI4	υ	Discrepancy	ь
		Result		Result		MASLOV	
		CEA2005		CEA2005			
				MASLOV			
001	1	1,00002	8	0,99936	8	-66	11
002	1	1,00435	8	1,00320	8	-115	11
005	1	1,00404	9	1,00376	9	-28	13
008	1	1,00170	9	1,00108	9	-62	13
009	1	0,99936	8	0,99881	8	-55	11
010	1	1,00255	8	1,00186	8	-69	11
011	1	0,99723	11	0,99735	10	12	15
012	1 s	1,00524	10	1,00418	10	-106	14
013	1 s	1,00644	11	1,00629	11	-15	16
014	1 s	1,00185	11	1,00166	11	-19	16
015	1 s	1,00230	11	1,00147	0	-83	11
018	1	0,98385	8	0,98393	8	8	11
					MIN	-115	
					MOY	-50	
					MAX	12	

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Concluding Remarks

✓ Benchmark experiments sensitive to the fission, capture cross sections, nu-bar and prompt neutron fission spectrum (PNFS)

✓ A right combination of capture-to-fission ratio (alpha) may lead to an improvement on the k_{eff} ;

✓ Further studies are needed using new PNFS evaluations;



Scheduled Work

✓ Finalize a document related to Benchmark list and calculations

✓ Few weeks of intensive work between CEA/ ORNL in 2012 on the evaluation benchmark calculations

✓ New PNFS evaluations to be tested (JEFF/ ENDF)

✓ Contributions from other Projects are welcomed