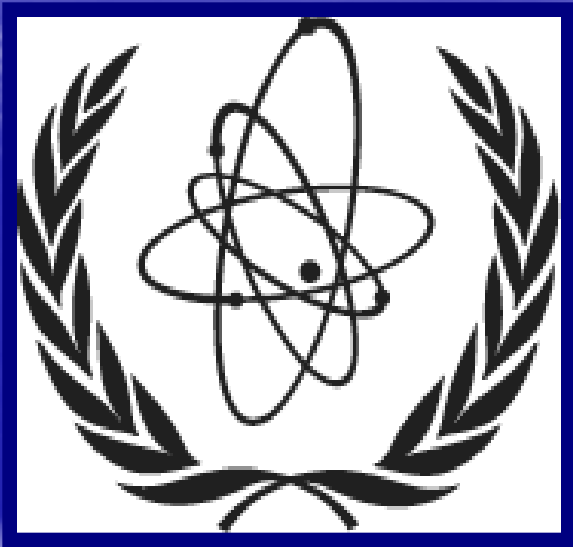


# Towards a new dosimetry library



R. Capote, M.A. Kellett and R.A. Forrest

IAEA/NAPC Nuclear Data Section

# Update of IRDF-2002 (release 2005)

- ❖ IAEA CM held in 2007, 2010:  
INDC(NDS)-0575 (2010) and  
INDC(NDS)-507 (2007)
- ❖ Zolotarev evaluations  
(partially supported by IAEA/NDS)  
INDC(NDS)-0526,  
INDC(NDS)-0546,  
INDC(NDS)-0584.
- ❖ IAEA Standards + ENDF/B-VII.0 + (cov)
- ❖ Model covariance calculations: TENDL  
Use for the extension of the energy range



	<b>Dosimetry reaction</b>	<b>E<sub>max</sub> [MeV]</b>	<b>Comments</b>
1	$^{24}\text{Mg}(n,p)$	21	Fission
2	$^{27}\text{Al}(n,p)$	40	Fission+Fusion
3	$^{27}\text{Al}(n,\alpha)$	40	Fission+Fusion
4	$^{32}\text{S}(n,p)$	21	Fission
5	$^{47}\text{Ti}(n,p)$	20	Fission
6	$^{55}\text{Mn}(n,2n)$	40	Fission+Fusion
7	$^{55}\text{Mn}(n,\gamma)$	20	Fission+Fusion
8	$^{57}\text{Fe}(n,\gamma)$	20	Fission
9	$^{58}\text{Fe}(n,\gamma)$	20	Fission
10	$^{59}\text{Co}(n,2n)$	60	Fission+Fusion
11	$^{59}\text{Co}(n,p)$	75	Fission+Fusion
12	$^{60}\text{Ni}(n,p)$	21	Fission
13	$^{63}\text{Cu}(n,2n)$	20	Fission
14	$^{64}\text{Zn}(n,p)$	20	Fission
15	$^{65}\text{Cu}(n,2n)$	20	Fission

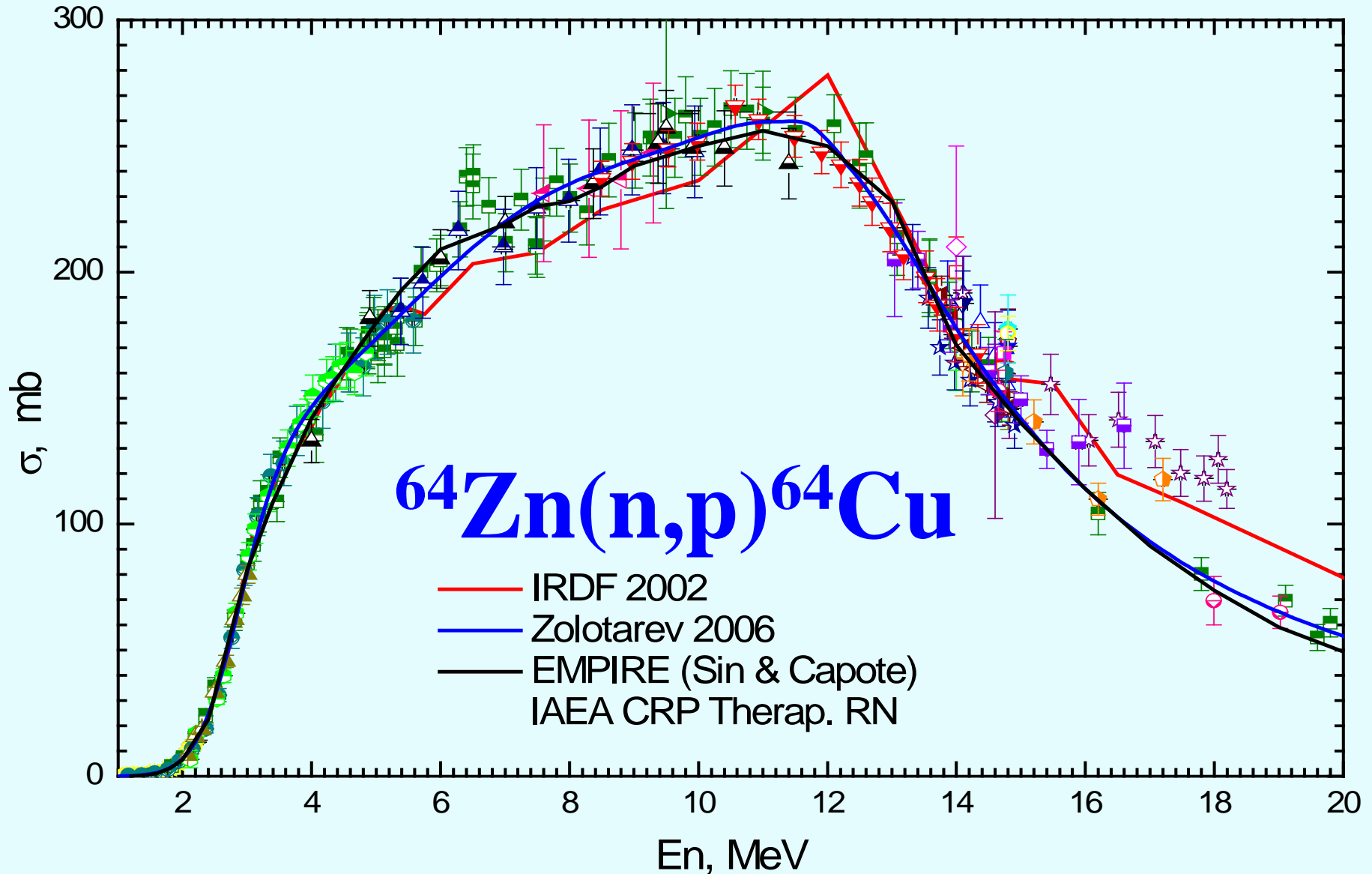
	<b>Dosimetry reaction</b>	<b>E<sub>max</sub> [MeV]</b>	<b>Comments</b>
16	$^{90}\text{Zr}(n,2n)$	40	Fission+Fusion
17	$^{115}\text{In}(n,2n)$	20	Fission+Fusion
18	$^{115}\text{In}(n,n')$	20	Fission+Fusion
19	$^{115}\text{In}(n,\gamma)$	20	Fission+Fusion
20	$^{127}\text{I}(n,2n)$	32	Fission
21	$^{197}\text{Au}(n,2n)$	40	Fission+Fusion
22	$^{197}\text{Au}(n,\gamma)$	30	Fission+Fusion
23	$^{199}\text{Hg}(n,n')^{199\text{m}}\text{Hg}$	20	Fission
24	$^{59}\text{Co}(n,3n)^{57}\text{Co}$	85	Fusion
25	$^{93}\text{Nb}(n,2n)^{92\text{m}}\text{Nb}$	40	Fission+Fusion
26	$^{89}\text{Y}(n,2n)^{88}\text{Y}$	40	Fission+Fusion
27	$^{169}\text{Tm}(n,2n)^{168}\text{Tm}$	40	Fission+Fusion
28	$^{209}\text{Bi}(n,3n)^{207}\text{Bi}$	45	Fusion



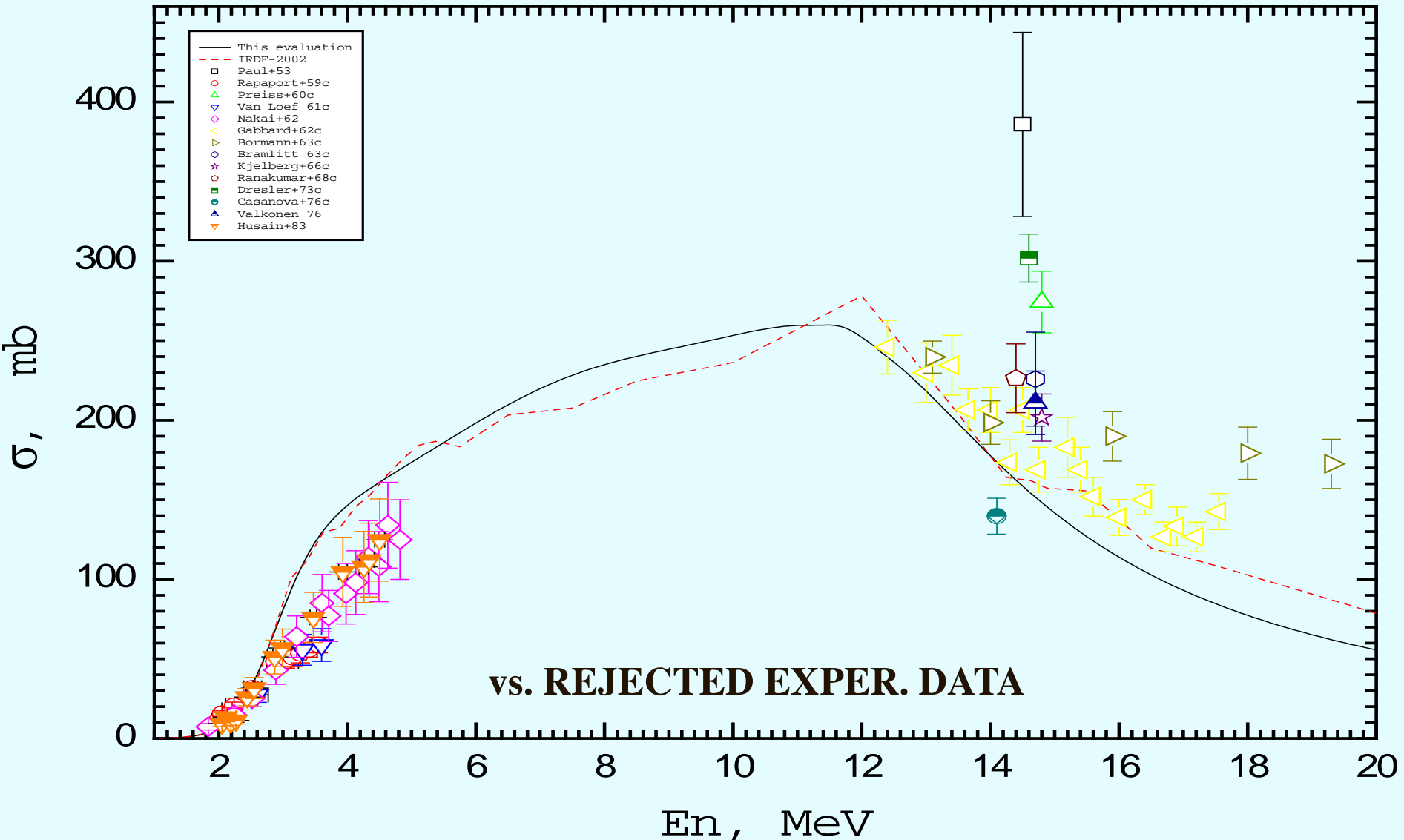
	<b>Dosimetry reaction</b>	<b>E<sub>max</sub> [MeV]</b>	<b>Comments</b>
29	$^{232}\text{Th}(n,f)$	60	Fission
30	$^{232}\text{Th}(n,\gamma)$	60	Fission
31	$^{235}\text{U}(n,f)$	200	Fission
32	$^{238}\text{U}(n,f)$	200	Fission
33	$^{238}\text{U}(n,\gamma)$	30	Fission
34	$^{239}\text{Pu}(n,f)$	200	Fission
35	$^{237}\text{Np}(n,f)$	30	Fission
36	$^6\text{Li}(n,t)$	20	Fission
37	$^{10}\text{B}(n,\alpha)$	20	Fission
	Cd-nat	20	Fission (no cov)



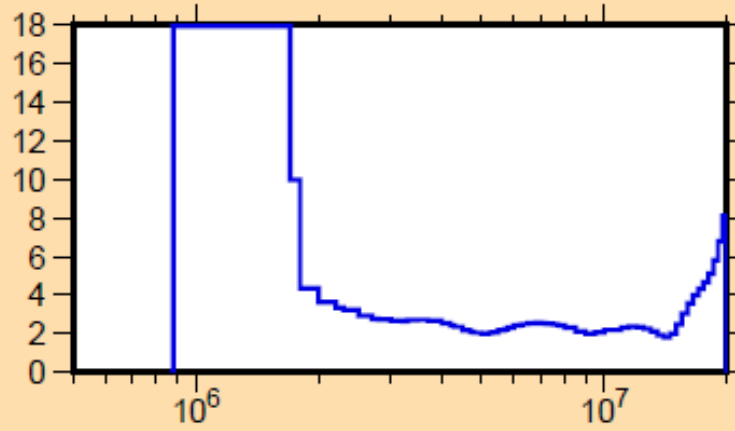
# New evaluation vs EMPIRE



# $^{64}\text{Zn}(n,p)^{64}\text{Cu}$

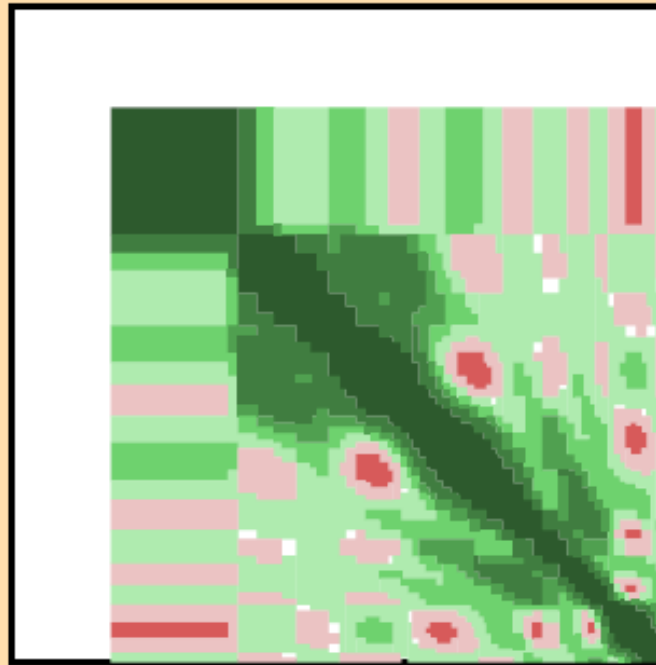


$\Delta\sigma/\sigma$  vs. E for  $^{64}\text{Zn}(n,p)$

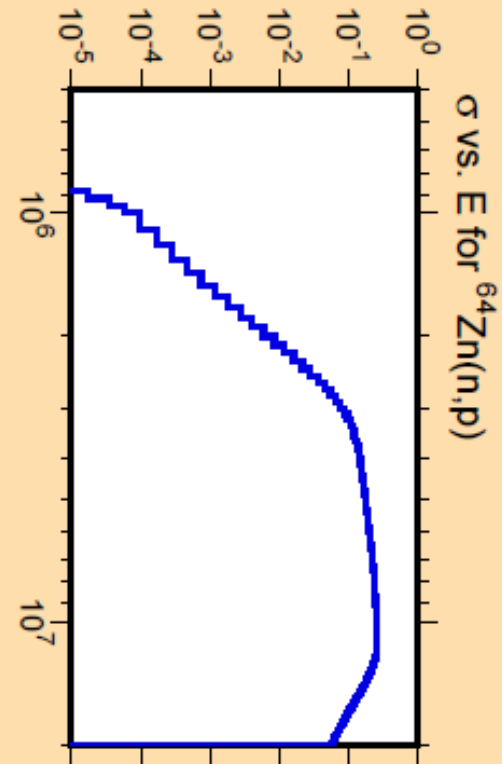


Ordinate scales are % relative standard deviation and bars.

Abscissa scales are energy (eV).



Correlation Matrix



# $^{64}\text{Zn}(n,p)^{64}\text{Cu}$

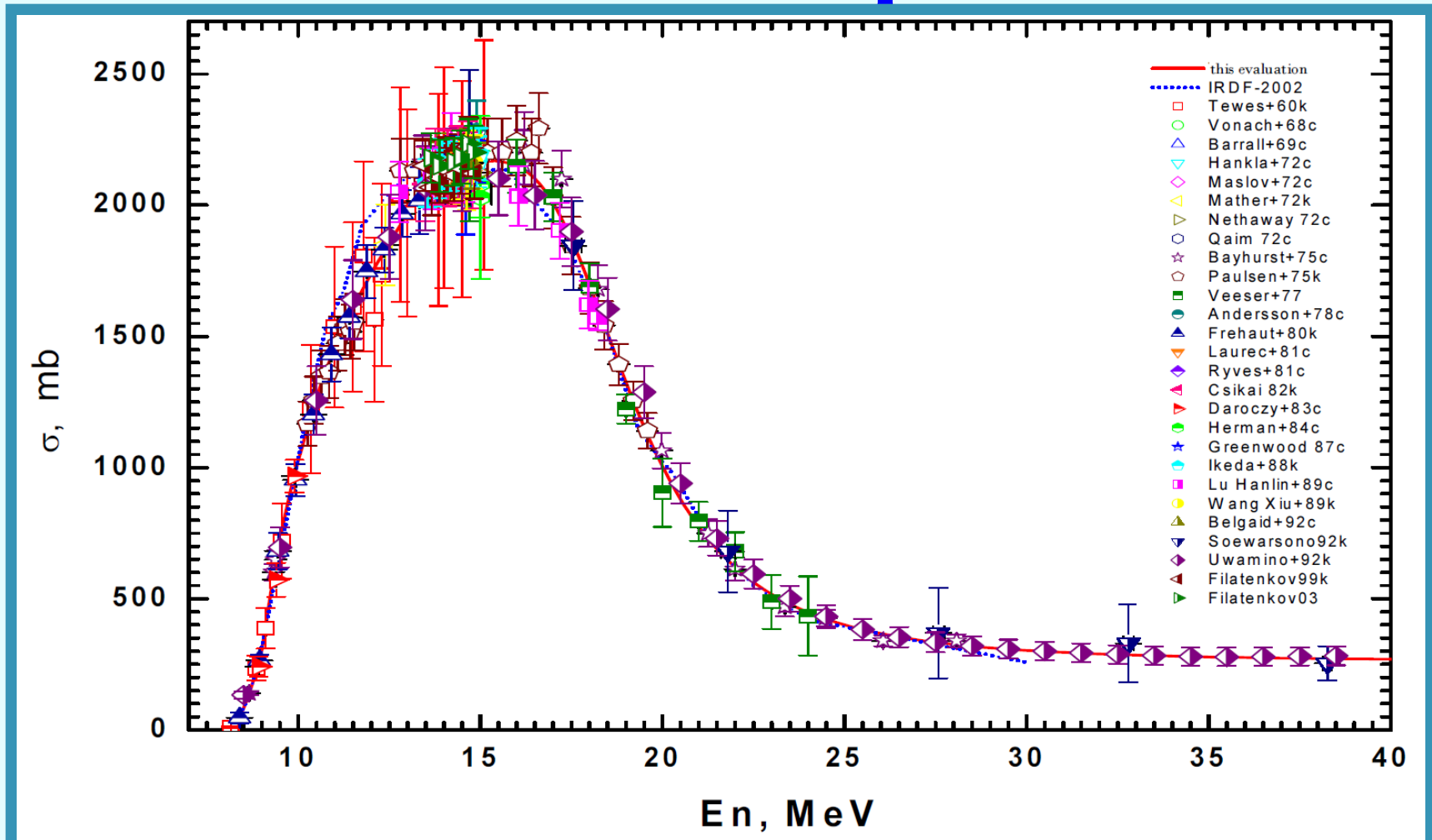
Type of neutron field	Average cross section, mb		C/E [**]
	Calculated	Measured	
$^{235}\text{U}$ thermal fission neutron spectrum	$38.9 \pm 0.7$ [A] 38.399 [B]	$38.9 \pm 2.8$ [**]	1.000 0.987
$^{252}\text{Cf}$ spontaneous fission neutron spectrum	$42.7 \pm 0.8$ [A] 42.095 [B]	$42.3 \pm 0.9$ [**]	1.009 0.994

[A] – Present evaluation;  
 [B] – IRDF-2002 (IRDF-90v.2);  
 [\*\*] – evaluated experimental cross section

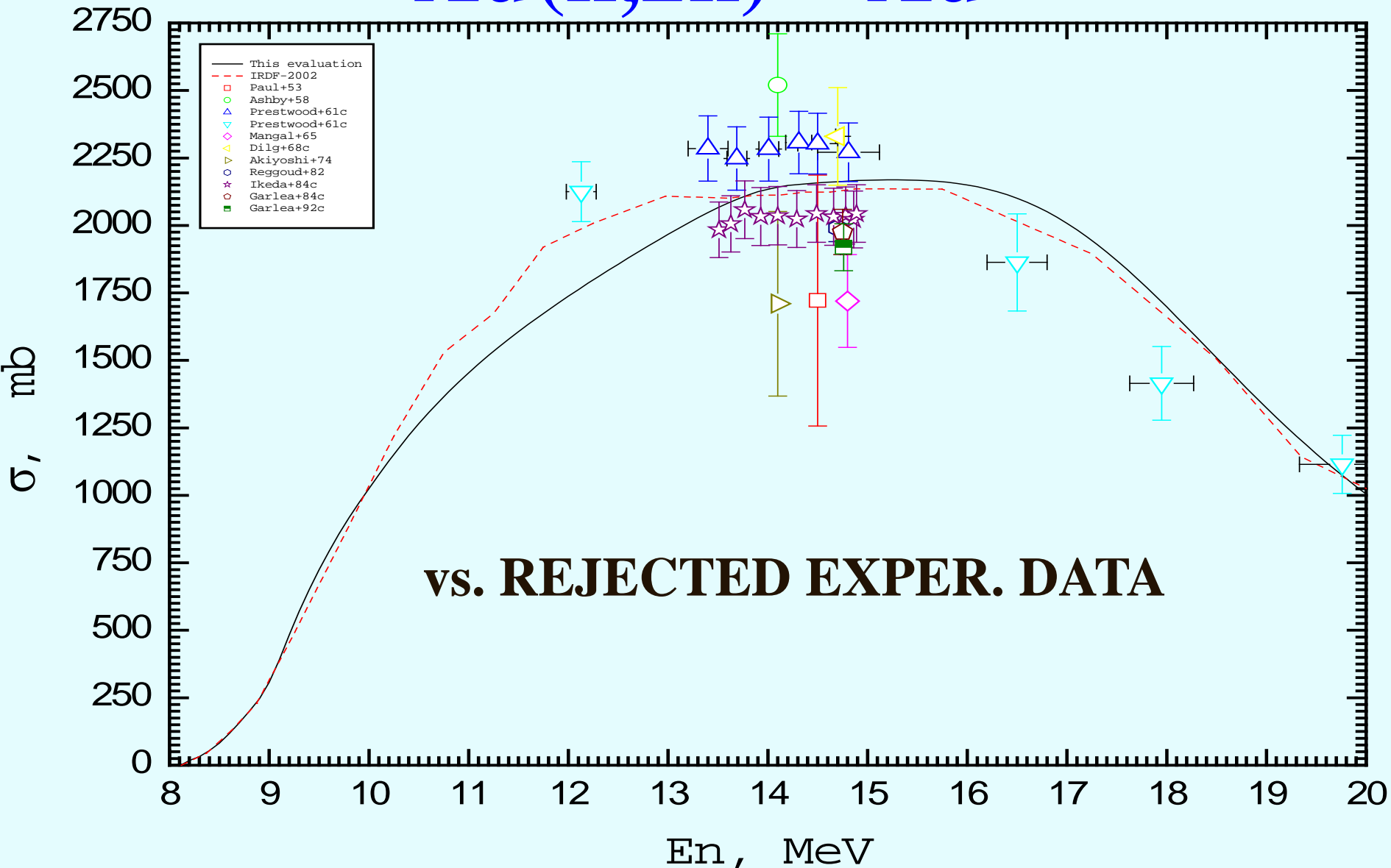


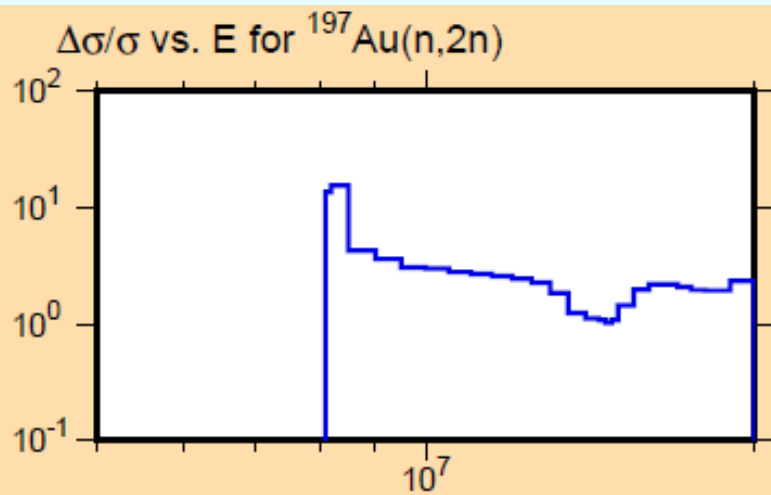


# $^{197}\text{Au}(n,2n)^{196}\text{Au}$ new evaluation vs selected exp. data



# $^{197}\text{Au}(n,2n)^{196}\text{Au}$

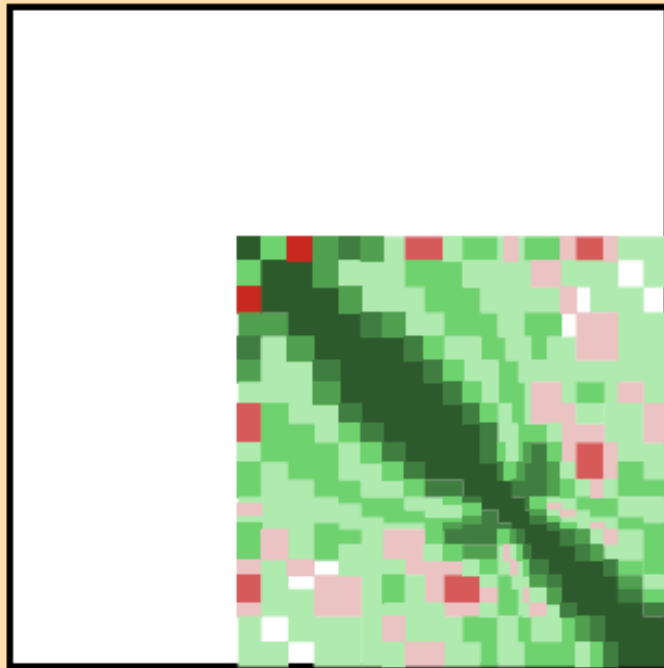




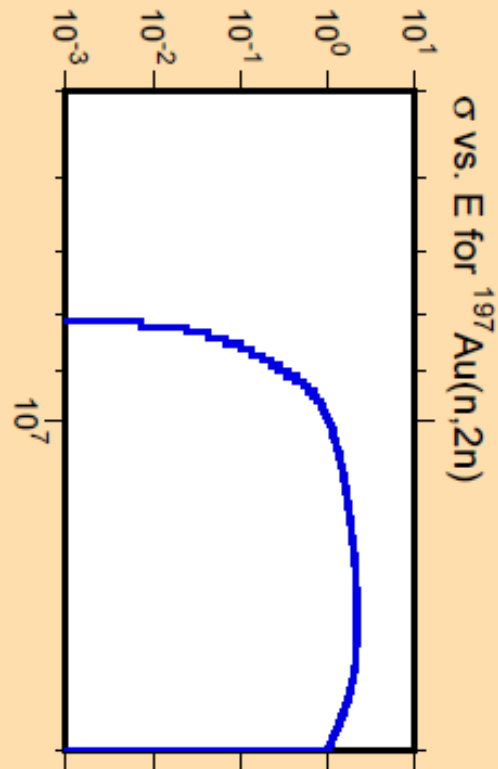
Ordinate scales are % relative standard deviation and bars.

Abscissa scales are energy (eV).

# $^{197}\text{Au}(n,2n)^{196}\text{Au}$

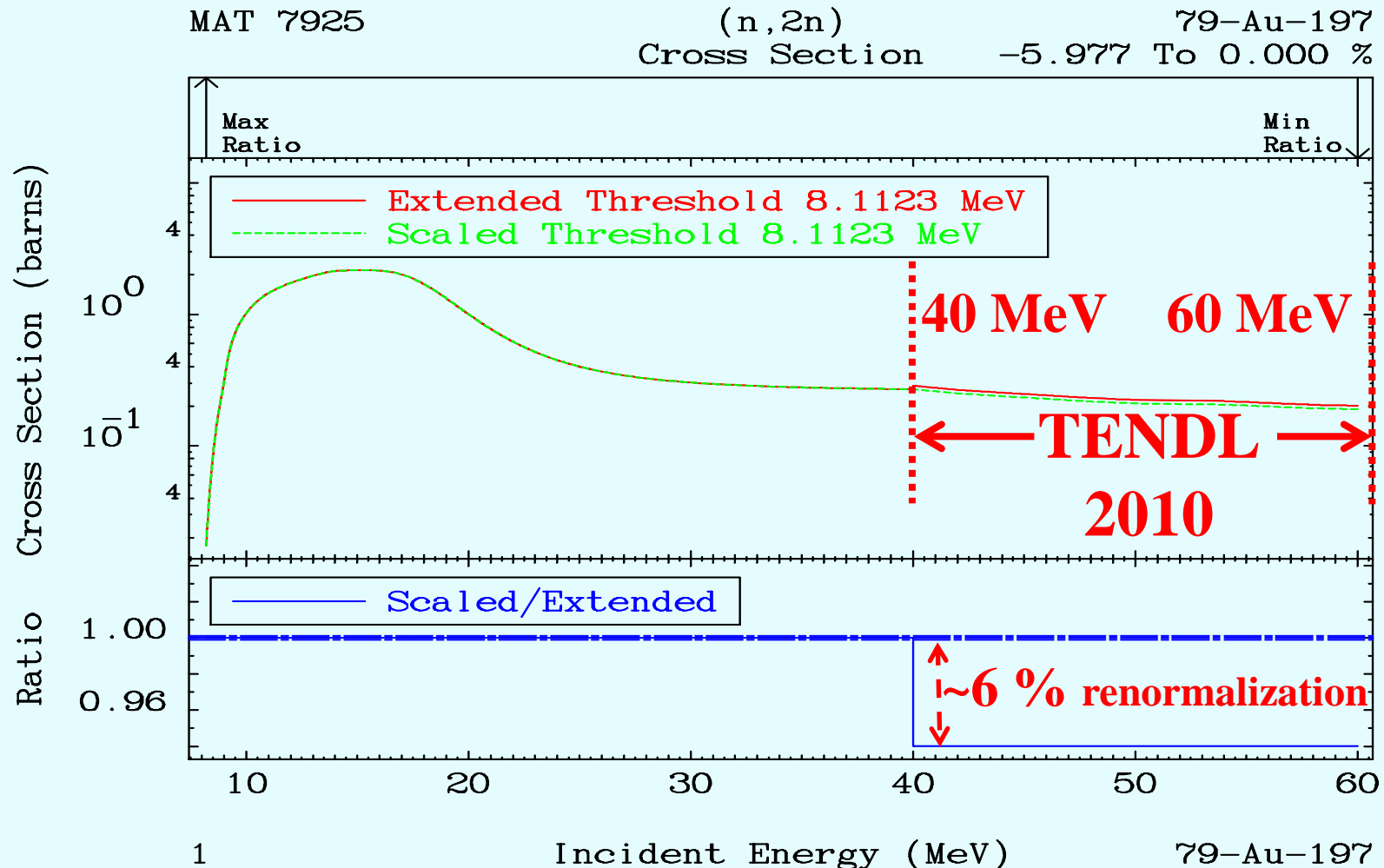


Correlation Matrix



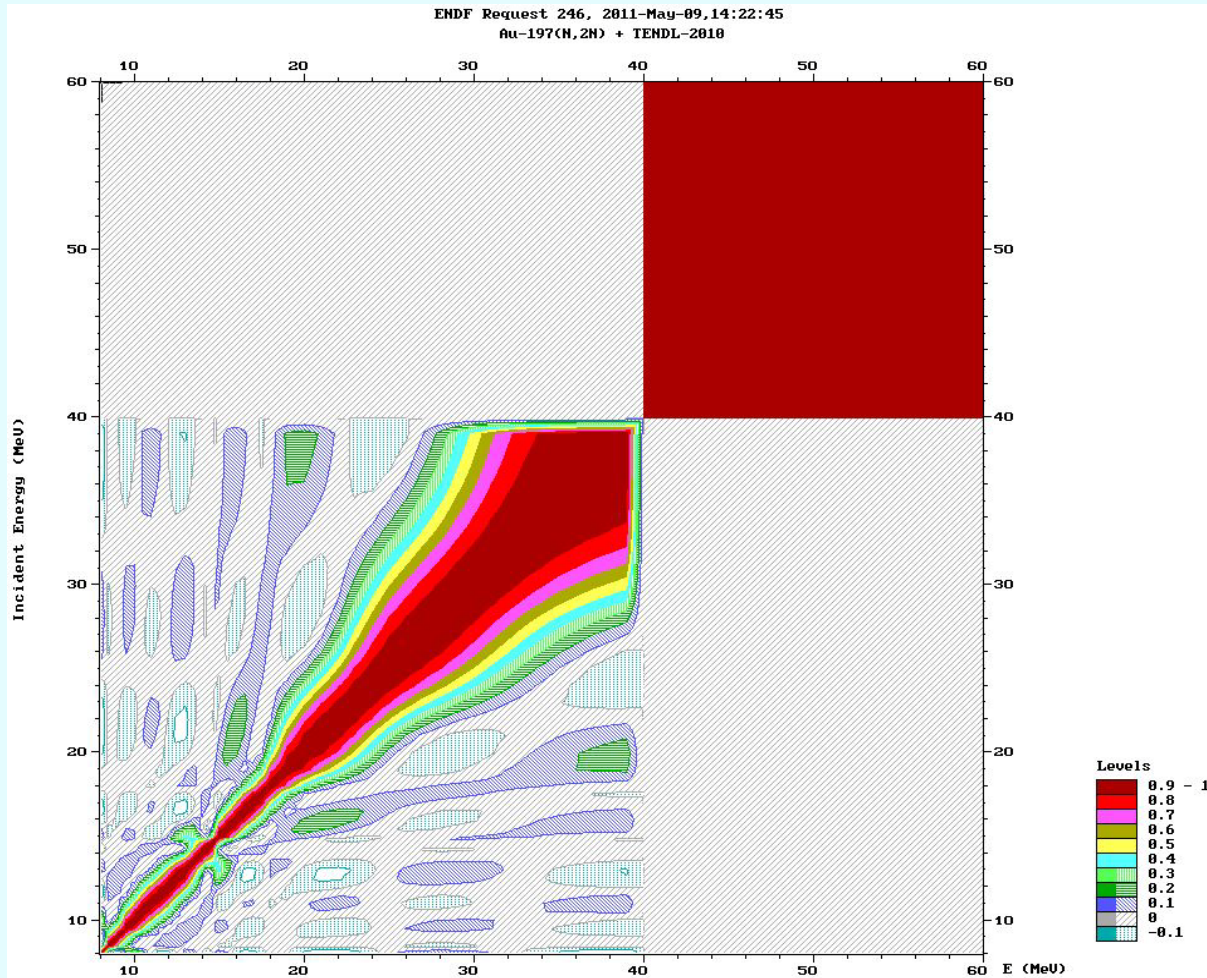
# Extension to 60 MeV – TENDL- 2010

## $^{197}\text{Au}(n,2n)^{196}\text{Au}$



# Extension to 60 MeV – TENDL- 2010

## $^{197}\text{Au}(n,2n)^{196}\text{Au}$

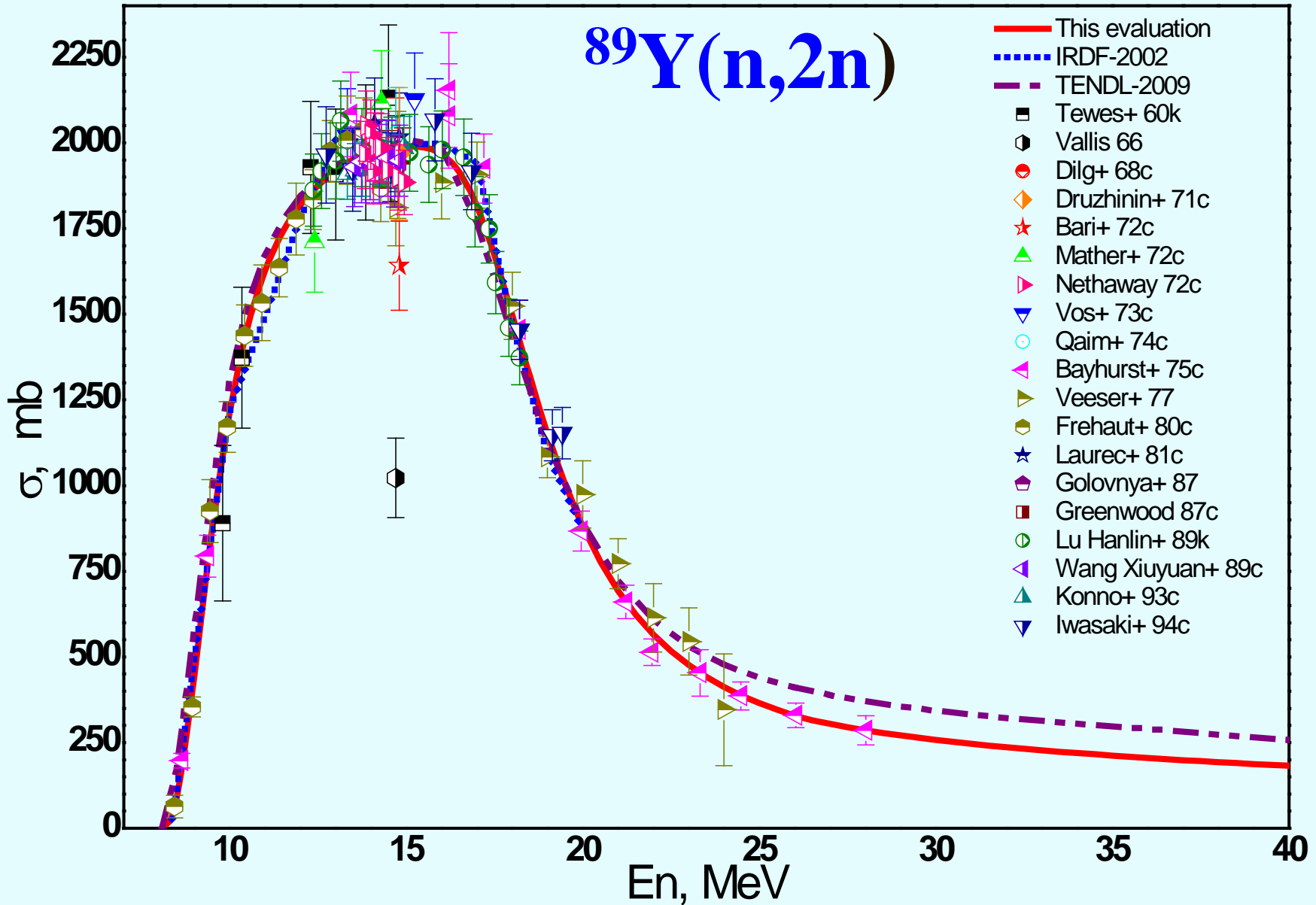


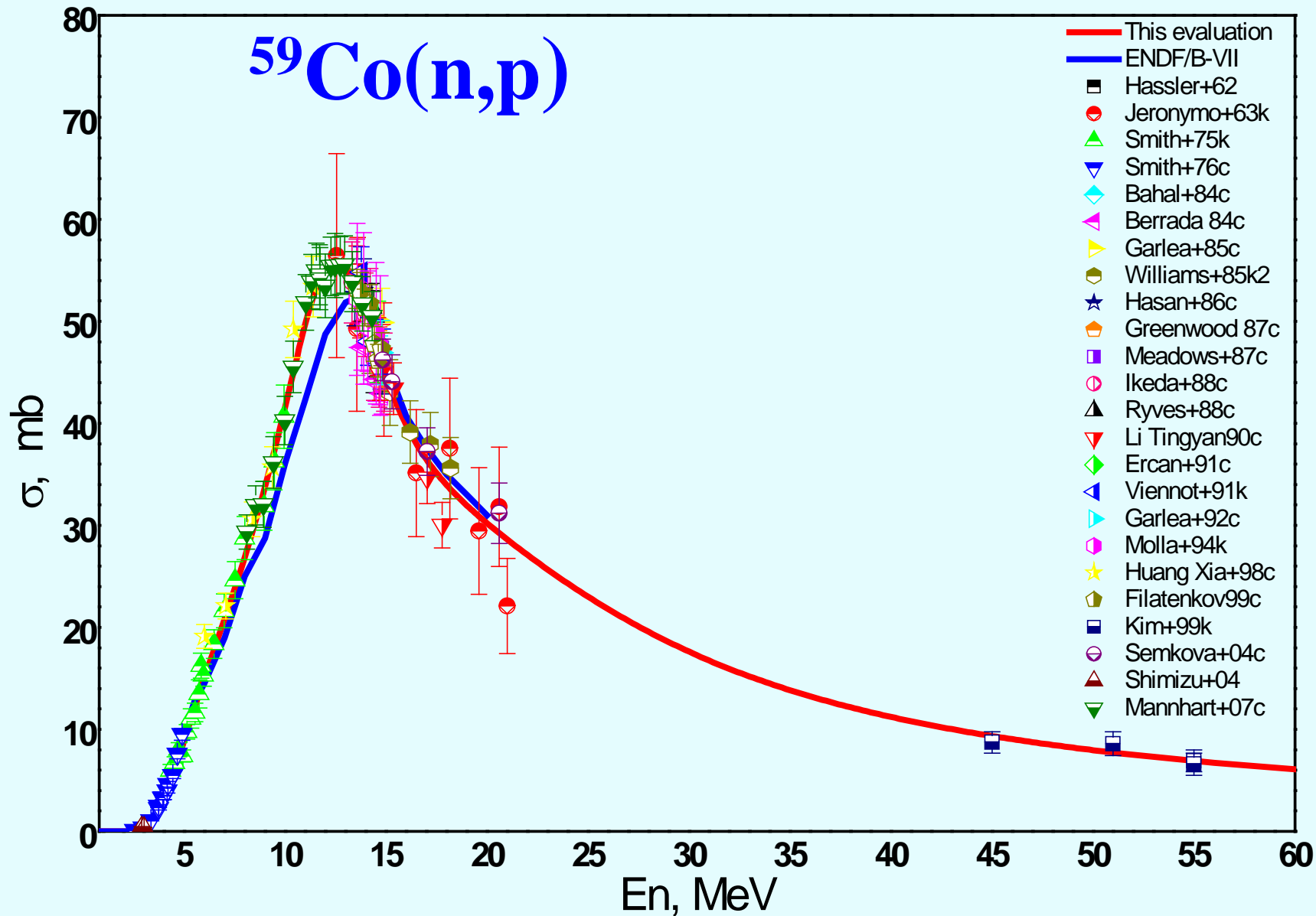
# $^{197}\text{Au}(n,2n)^{196}\text{Au}$

Type of neutron field	Average cross section, mb		C/E
	Calculated	Measured	
$^{235}\text{U}$ thermal fission neutron spectrum	$3.33 \pm 0.07$ [A] 3.4762 [B]	$3.392 \pm 0.080$	0.988 1.025
$^{252}\text{Cf}$ spontaneous fission neutron spectrum	$5.53 \pm 0.15$ [A] 5.7469 [B]	$5.506 \pm 0.101$	1.004 1.044

[A] – Present evaluation;  
 [B] – IRDF-2002 (IRDF-90v.2).

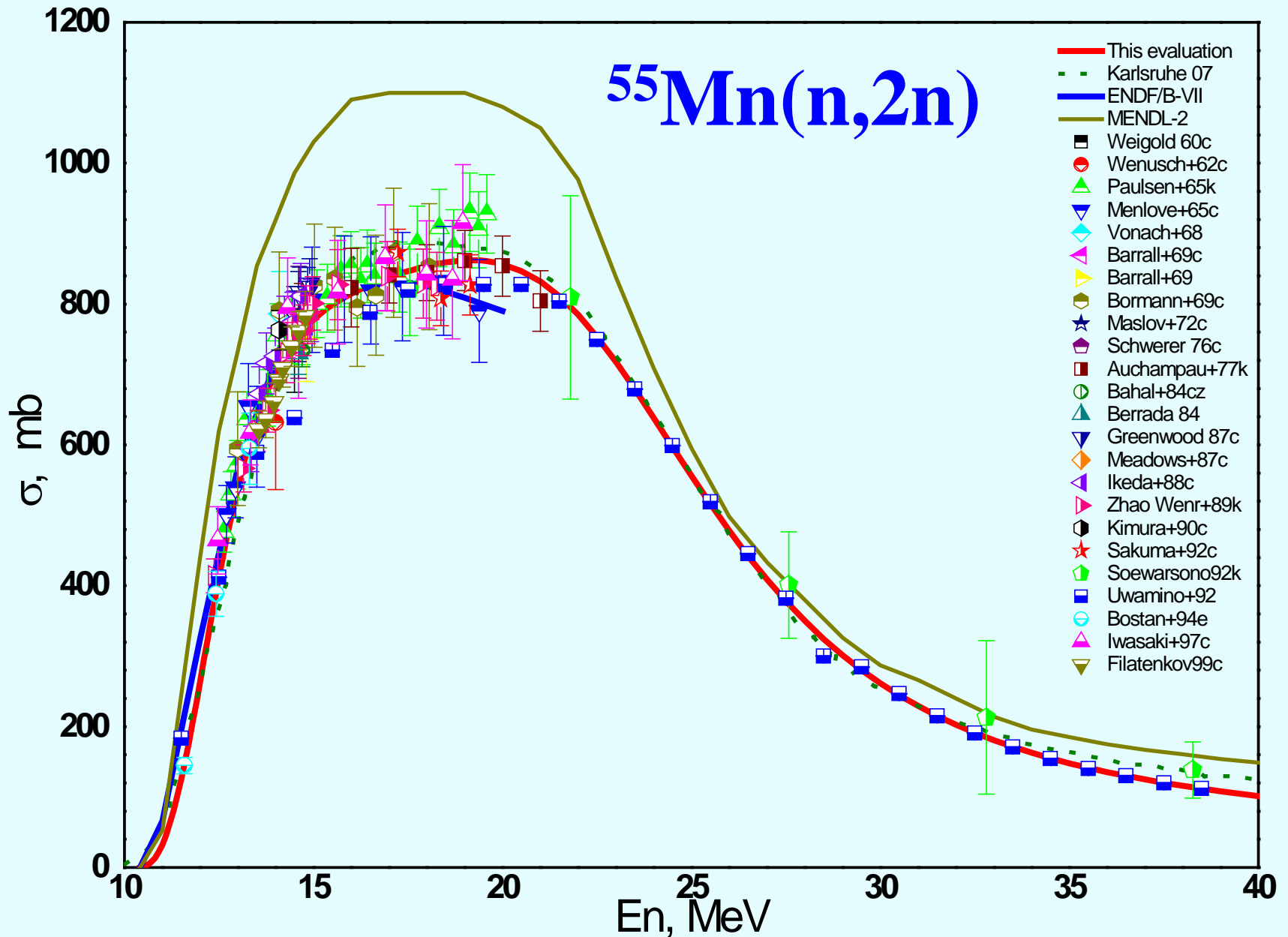


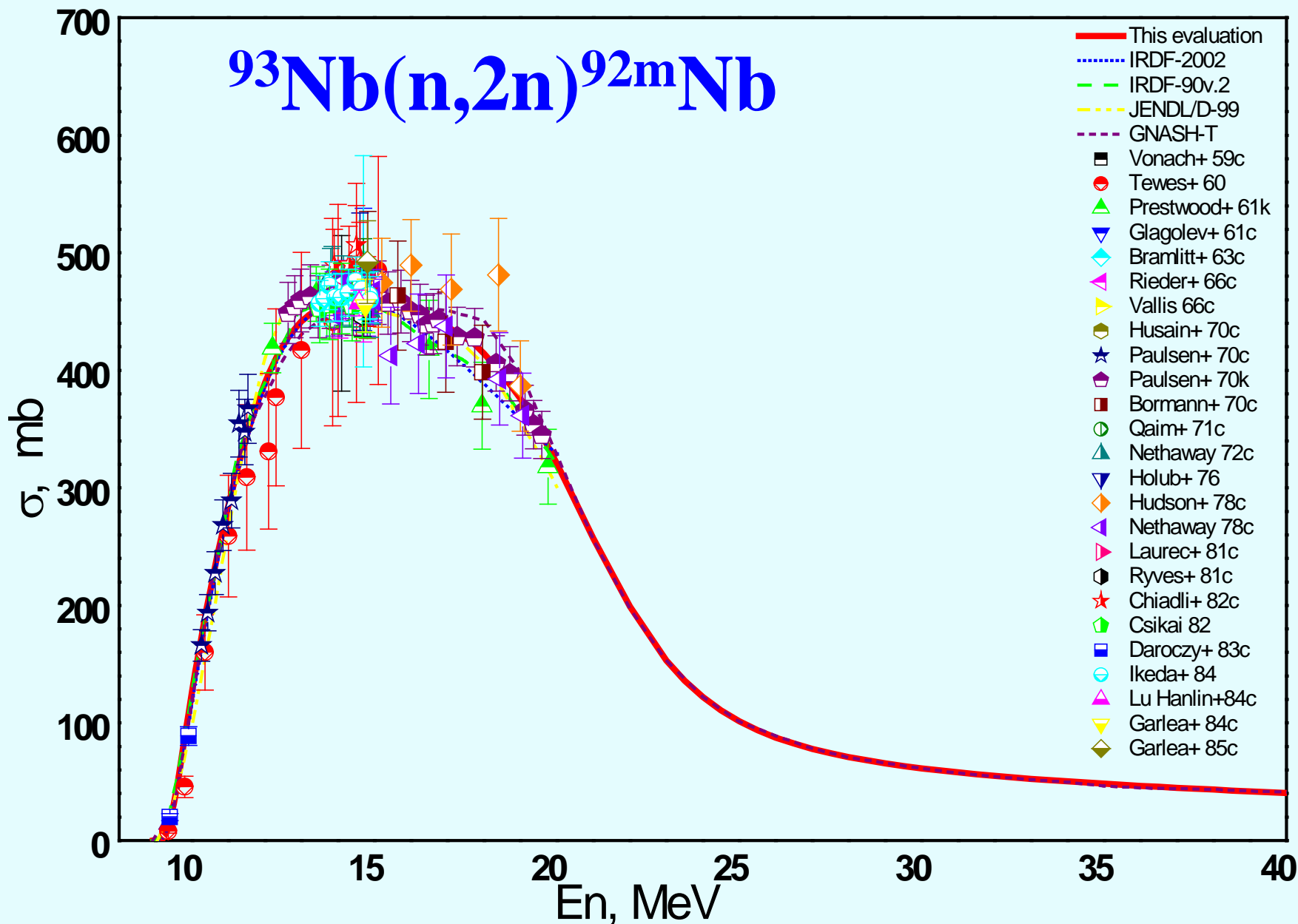






# $^{55}\text{Mn}(n,2n)$





UPDATED REACTIONS	$E_{50\%}$ (MeV)	Calculated $\langle\sigma\rangle$ , mb	Experiment $\langle\sigma\rangle$ , mb	C/E	Calculated $\langle\sigma\rangle$ , mb	Experiment $\langle\sigma\rangle$ , mb	C/E
${}^6\text{Li}(n,t){}^4\text{He}$ ENDF/B-VII.1 $\beta$ 2	0.66	$323.2 \pm 2.1$ (0.6%)	--	--	$321.3 \pm 2.8$ (0.9%)	--	--
${}^{197}\text{Au}(n,\gamma){}^{198}\text{Au}$	0.72	$75.8 \pm 0.4$ (0.6%)	$74.0 \pm 3.0$ (4.0%)	1.024	$74.9 \pm 0.7$ (0.9%)	$76.8 \pm 1.2$ (1.59%)	0.975
${}^{58}\text{Fe}(n,\gamma){}^{58}\text{Fe}$ JEFF 3.1	0.72	$2.0 \pm 0.2$ (10%)	--	--	$2.0 \pm 0.2$ (10%)	--	--
${}^{10}\text{B}(n,\alpha_0+\alpha_1)$ ENDF/B-VII.1 $\beta$ 2	0.84	$447 \pm 8$ (1.8%)	--	--	$446 \pm 7$ (1.6%)	--	--
${}^{232}\text{Th}(n,\gamma){}^{233}\text{Th}$ ENDF/B-VII.0	0.88	$91.9 \pm 2.6$ (2.8%)	--	--	$90.0 \pm 2.6$ (2.8%)	$[87.8 \pm 4.0]$ (4.6%)	1.025
${}^{238}\text{U}(n,\gamma){}^{239}\text{U}$	0.88	$69.0 \pm 0.7$ (1.0%)	--	--	$67.5 \pm 0.7$ (1.0%)	--	--
${}^{55}\text{Mn}(n,\gamma){}^{56}\text{Mn}$	0.88	<b><math>2.60 \pm 0.25</math></b> <b>(10%)</b>	--	--	<b><math>2.60 \pm 0.25</math></b> <b>(10%)</b>	<b><math>[2.96 \pm 0.21]</math></b> <b>(7%)</b>	<b>0.878</b>
${}^{235}\text{U}(n,f)$	1.70	$1222 \pm 5$ (0.4%)	$1217 \pm 14$ (1.1%)	1.004	$1225 \pm 5$ (0.4%)	$1210 \pm 14$ (1.20%)	1.012
${}^{239}\text{Pu}(n,f)$	1.70	$1795 \pm 9$ (0.5%)	$1831 \pm 32$ (1.7%)	0.980	$1796 \pm 9$ (0.5%)	$1812 \pm 25$ (1.37%)	0.991
${}^{237}\text{Np}(n,f)$	2.00	$1356 \pm 22$ (1.7%)	$1350 \pm 24$ (1.8%)	1.004	$1360 \pm 23$ (1.7%)	$1361 \pm 22$ (1.59%)	0.999
${}^{237}\text{Np}(n,f)$ - ENDF/B-VII.1 $\beta$ 2	2.00	$1354 \pm 36$ (2.7%)	$1350 \pm 24$ (1.8%)	1.003	$1359 \pm 36$ (2.7%)	$1361 \pm 22$ (1.59%)	0.998



UPDATED REACTIONS	$E_{50\%}$ (MeV)	Calculated $\langle\sigma\rangle$ , mb	Experiment $\langle\sigma\rangle$ , mb	C/E	Calculated $\langle\sigma\rangle$ , mb	Experiment $\langle\sigma\rangle$ , mb	C/E
$^{115}\text{In}(n,n')^{115\text{m}}\text{In}$	2.60	$188.4 \pm 3.2$ (1.7%)	$187.8 \pm 2.3$ (1.2%)	1.003	$191.8 \pm 3.3$ (1.7%)	$197.4 \pm 2.7$ (1.37%)	0.972
$^{238}\text{U}(n,f)$	2.70	$309.2 \pm 1.6$ (0.5%)	$309.4 \pm 3.5$ (1.1%)	0.999	$318.5 \pm 2.1$ (0.6%)	$325.7 \pm 5.3$ (1.64%)	0.978
$^{99}\text{Hg}(n,n')^{199\text{m}}\text{Hg}$	3.00	$286 \pm 10$ (3.7%)	$278 \pm 16$ (5.3%)	1.029	$296 \pm 11$ (3.6%)	$298 \pm 5$ (1.81%)	0.993
$^{232}\text{Th}(n,f)$ ENDF/B-VII.0	3.00	$74.1 \pm 1.5$ (2.1%)	$74.5 \pm 3.1$ (4.2%)	0.996	$77.6 \pm 1.6$ (2.1%)	$76.9 \pm 2.9$ (3.8%) <b>[ 89.4 ± 2.7 ]</b> <b>(3.0%)</b>	1.009 <b>0.868</b>
$^{47}\text{Ti}(n,p)^{47}\text{Sc}$	3.80	$18.1 \pm 0.5$ (2.8%)	$18.0 \pm 0.8$ (4.6%)	1.005	$19.56 \pm 0.55$ (2.8%)	$19.27 \pm 0.32$ (1.66%)	1.015
$^{32}\text{S}(n,p)^{32}\text{P}$	4.00	$68.2 \pm 1.7$ (2.5%)	$69.1 \pm 1.4$ (2.0%)	0.987	$74.1 \pm 1.90$ (2.6%)	$72.5 \pm 2.5$ (3.49%)	1.022
$^{64}\text{Zn}(n,p)^{64}\text{Cu}$	4.10	$38.9 \pm 0.7$ (1.7%)	$38.6 \pm 1.6$ (4.3%)	1.008	$42.7 \pm 0.80$ (1.9%)	$42.2 \pm 1.0$ (2.3%) <sup>4</sup>	1.012
$^{27}\text{Al}(n,p)^{27}\text{Mg}$	5.80	$3.96 \pm 0.08$ (2.%)	$3.90 \pm 0.07$ (1.8%)	1.015	$4.75 \pm 0.11$ (2.4%)	$4.88 \pm 0.10$ (2.14%)	0.973
$^{59}\text{Co}(n,p)^{59}\text{Fe}$	5.90	$1.41 \pm 0.05$ (3.6%)	$1.40 \pm 0.03$ (2.4%)	1.007	$1.72 \pm 0.06$ (3.6%)	$1.69 \pm 0.04$ (2.48%)	1.017
$^{60}\text{Ni}(n,p)^{60}\text{Co}$	7.00	$2.17 \pm 0.04$ (2.0%)	$2.18 \pm 0.10$ (4.8%)	0.995	$2.80 \pm 0.06$ (2.3%)	<b>[2.180 ± .104]</b> <b>(4.7%)</b>	<b>1.171</b>



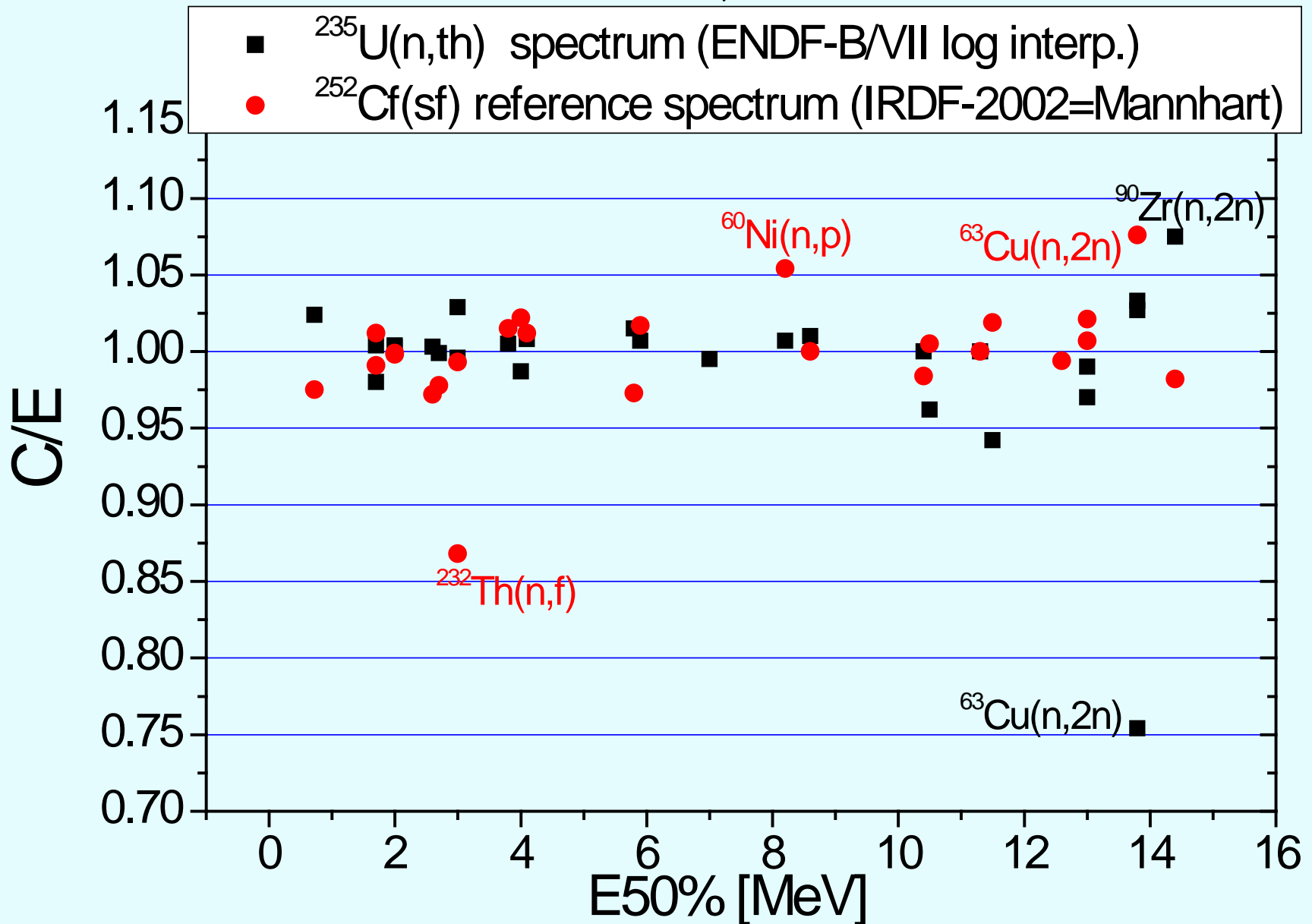
<b>UPDATED REACTIONS</b>	<b>E<sub>50%</sub> (MeV)</b>	<b>Calculated &lt;σ&gt;, mb</b>	<b>Experiment &lt;σ&gt;, mb</b>	<b>C/E</b>	<b>Calculated &lt;σ&gt;, mb</b>	<b>Experiment &lt;σ&gt;, mb</b>	<b>C/E</b>
<sup>24</sup> Mg(n,p) <sup>24</sup> Na	8.20	1.50 ± 0.01 (0.8%)	1.49 ± 0.03 (1.8%)	1.007	2.10 ± 0.04 (1.8%)	1.996 ± 0.049 (2.44%)	1.054
<sup>27</sup> Al(n,α) <sup>24</sup> Na	8.60	0.707 ± 0.005 (0.7%)	0.701 ± 0.009 (1.3%)	1.010	1.02 ± 0.02 (1.8%)	1.02 ± 0.01 (1.28%)	1.000
<sup>169</sup> Tm(n,2n) <sup>168</sup> Tm	10.4	3.74 ± .10 (2.6%)	3.74 ± .16 (4.2%)	1.000	6.3 ± 0.2 (3.1%)	6.4 ± 0.4 (6.4%)	0.984
<sup>197</sup> Au(n,2n) <sup>196</sup> Au	10.5	3.33 ± 0.06 (2.0%)	3.39 ± 0.08 (5.1%)	0.962	5.531 ± 0.15 (2.8%)	5.506 ± 0.101 (1.83%)	1.005
<sup>93</sup> Nb(n,2n) <sup>92m</sup> Nb	11.3	0.435 ± 0.004 (0.9%)	0.435 ± 0.010 (2.3%)	1.000	.791 ± 0.019 (2.4%)	.791 ± 0.035 (4.5%)	1.000



UPDATED REACTIONS	E <sub>50%</sub> (MeV)	Calculated <σ>, mb	Experiment <σ>, mb	C/E	Calculated <σ>, mb	Experiment <σ>, mb	C/E
<sup>127</sup> I(n,2n) <sup>126</sup> I	11.5	1.13 ± 0.04 (3.2%)	1.20 ± 0.04 (3.4%)	0.942	2.11 ± 0.080 (3.8%)	2.07 ± 0.06 (2.73%)	1.019
<sup>65</sup> Cu(n,2n) <sup>64</sup> Cu	12.6	0.322 ± 0.006 (2%)	--	--	0.654 ± 0.022 (3.5%)	0.658 ± 0.015 (2.22%)	0.994
<sup>55</sup> Mn(n,2n) <sup>54</sup> Mn	13.0	0.198 ± 0.004 (2.0%)	0.20 ± 0.07 (3.6%)	0.990	0.420 ± 0.020 (3.7%)	0.407 ± 0.009 (2.33%)	1.021
<sup>59</sup> Co (n,2n) <sup>58</sup> Co	13.0	0.194 ± 0.003 (1.7%)	0.20 ± 0.08 (2.5%)	0.970	0.408 ± 0.020 (3.6%)	0.405 ± 0.010 (2.51%)	1.007
<sup>89</sup> Y(n,2n) <sup>88</sup> Y	13.8	0.154 ± 0.002 (1.3%)	0.150 ± 0.005 (3.3%)	1.027	0.346 ± 0.015 (4.4%)	--	--
<sup>89</sup> Y(n,2n) <sup>88</sup> Y - ENDF/B-VII.0	13.8	0.155 ± 0.004 (2.7%)	0.150 ± 0.005 (3.3%)	1.033	0.349 ± 0.017 (5.0%)	--	--
<sup>63</sup> Cu(n,2n) <sup>62</sup> Cu	13.8	<b>0.089 ± 0.001</b> <b>(1.5%)</b>	<b>0.118 ± 0.007</b> <b>(5.9%)</b>	<b>0.754</b>	<b>0.198 ± 0.009</b> <b>(4.4%)</b>	<b>0.184 ± 0.007</b> <b>(3.98%)</b>	<b>1.076</b>
<sup>90</sup> Zr(n,2n) <sup>89</sup> Zr	14.4	0.092 ± 0.009 (.9%)	0.086 ± .005 (5.8%) 0.103 ± 0.003 (2.7%)	1.075 0.893	0.217 ± 0.010 (5.2%)	0.221 ± 0.006 (2.89%)	0.982



# IRDFF extension, 37 new evaluations



# Conclusions

- ❑ International Reactor Dosimetry File IRDF-2002 released in 2005 [1]; library contains 66 dosimetry reactions. Update is needed.
- ❑ New suitable evaluations including covariances became available
  - IAEA neutron cross-section standards [2] combined with ENDF-B/VII.1 evaluations [3],
  - Zolotarev evaluations [4-6].
- ❑ 37 dosimetry reactions updated and compared to measurements in ENDF-B/VII.0  $^{235}\text{U}$  thermal and  $^{252}\text{Cf}$  fast neutron fields.
- ❑ Formal extension of all evaluations up to 60 MeV of incident neutron energy using TENDL-2010 [7] done.
- ❑ **New IAEA dosimetry library will be released for testing in December 2011**

## REFERENCES:

- [1] O. Bersillon *et al.*, IRDF-2002, International Atomic Energy Agency, Vienna, Austria, TRS 452 (2006).
- [2] A. D. Carlson, *et al.*, *Nucl. Data Sheets* 110 (2009) 3215-3324.
- [3] L. Leal, Evaluations in the ENDF/B-VII.1 library with covariances, *priv. comm.*, 2010.
- [4] K. I. Zolotarev, INDC(NDS)-0526, IAEA, Vienna, August 2008.
- [5] K. I. Zolotarev, INDC(NDS)-0546, IAEA, Vienna, April 2009.
- [6] K. I. Zolotarev, INDC(NDS)-0584, IAEA, Vienna, November 2010.
- [7] A. J. Koning and D. Rochman, TENDL-2010, The Netherlands.





# Acknowledgements

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