

LANL Data Testing

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Introduction

■ ENDF/B-VII.1 β 0 created last week.

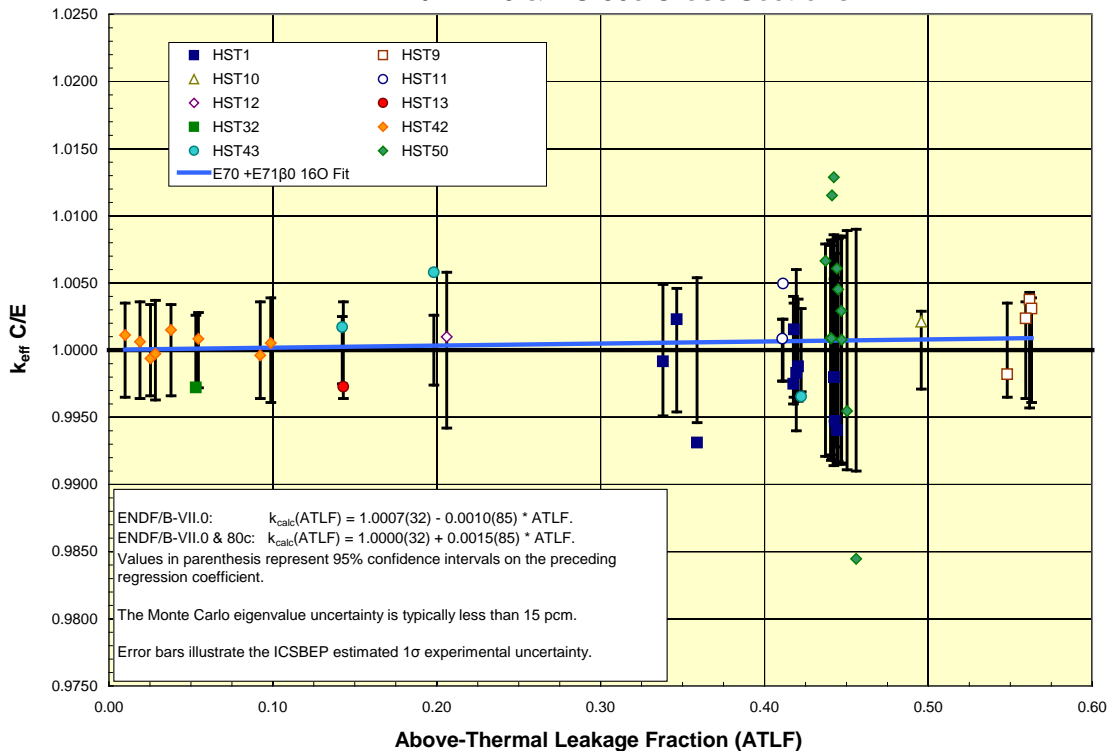
- Limited new testing to date
 - ^{16}O (Hale)
 - ^{90}Ti (ORNL/LANL)
 - ^{180}W (IAEA; includes ^{180}W)

■ Actinides ($^{236,237}\text{U}$, ^{237}Np , ^{240}Pu , ^{241}Am)

- Improved reaction rate ratios observed in various LANL assemblies (Godiva, Flattop, Big-10).
 - See November, 2009 and June, 2010 presentations.

Data Testing: E71 β 0 ¹⁶O

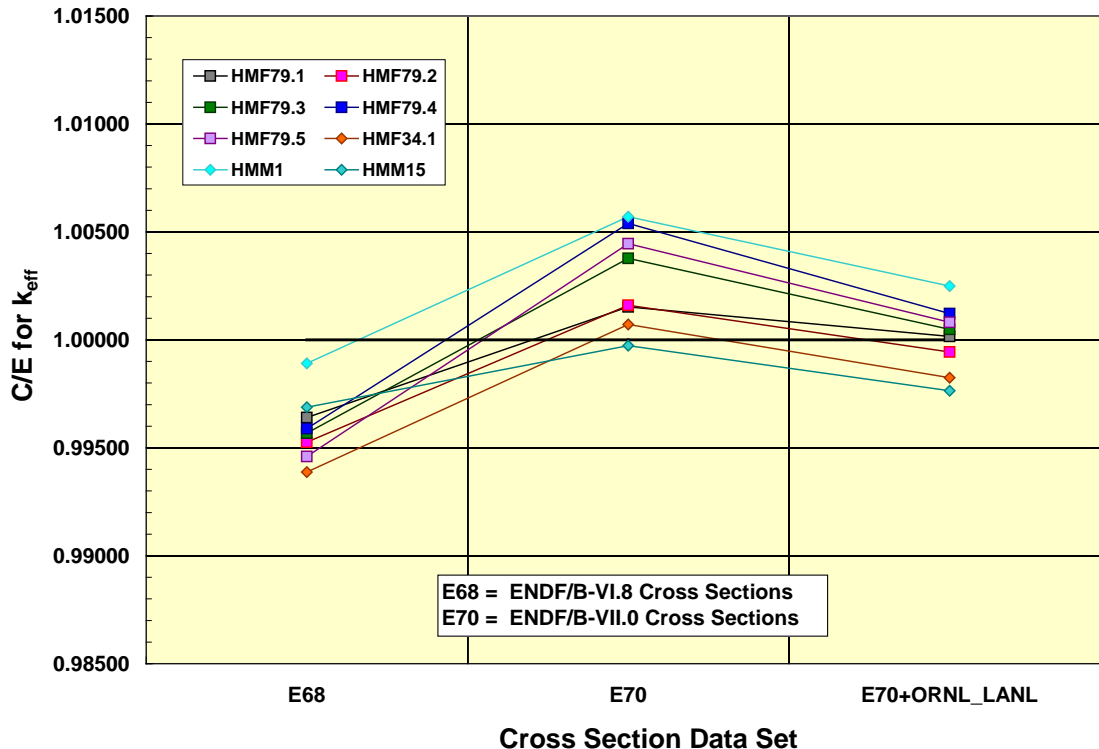
Calculated HEU-SOL-THERM-xxx Eigenvalues with
ENDF/B-VII.0 & ¹⁶O 80c Cross Sections



- Small improvement in regression intercept
- Small decrement in regression slope.
- Both intercept and slope were, and remain, statistically equal to unity and zero respectively.

Data Testing: Ti Benchmarks

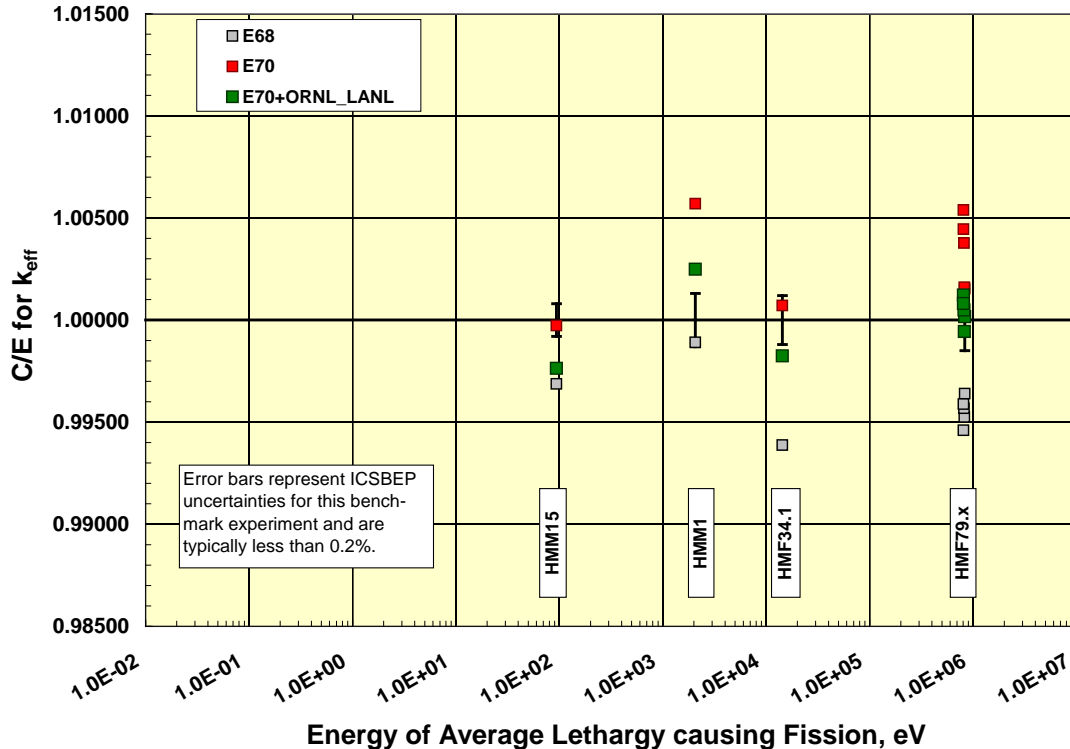
Calculated ICSBEP Benchmark Eigenvalues
with Various Cross Section Data Sets



- ENDF/B-VII.0 based eigenvalues are less accurate than those obtained with ENDF/B-VI.8.
- Revised ORNL/LANL Ti isotopic data sets eliminate much of this deficiency.

Data Testing: Ti Benchmarks

Calculated ICSBEP Benchmark Eigenvalues
with Various Cross Section Data Sets



- Calculated eigenvalues show no trend with average energy.

Data Testing: W Benchmarks

■ Isotopic W Evaluations from the IAEA

- Not neutronically important, but includes ^{180}W
- Limited criticality testing to date
 - HMF3.8 to 3.11: 1.9” to 6.5” thick spherical reflector
 - PMF5: 1.85” thick spherical reflector

■ ENDF/B-VII.0 Calculated Eigenvalues are Too High

- HMF3.8 3.11 = 1.00841(9), 1.00930(9), 1.01262(9) and 1.01677(9)
- PMF5 = 1.00923(9)

■ ENDF/B-VII.1 β_0 Calculated Eigenvalues are ~750 pcm Lower, but the Increasing Trend of k_{calc} with Increased Amounts of W Remains.

- HMF3.8 – 3.11 = 1.00124(9), 1.00174(9), 1.00519(9) and 1.00978(9)
- PMF5 = 1.00068(9)

■ Additional W Benchmarks are now in the ICSBEP Handbook

Future Data Testing Work

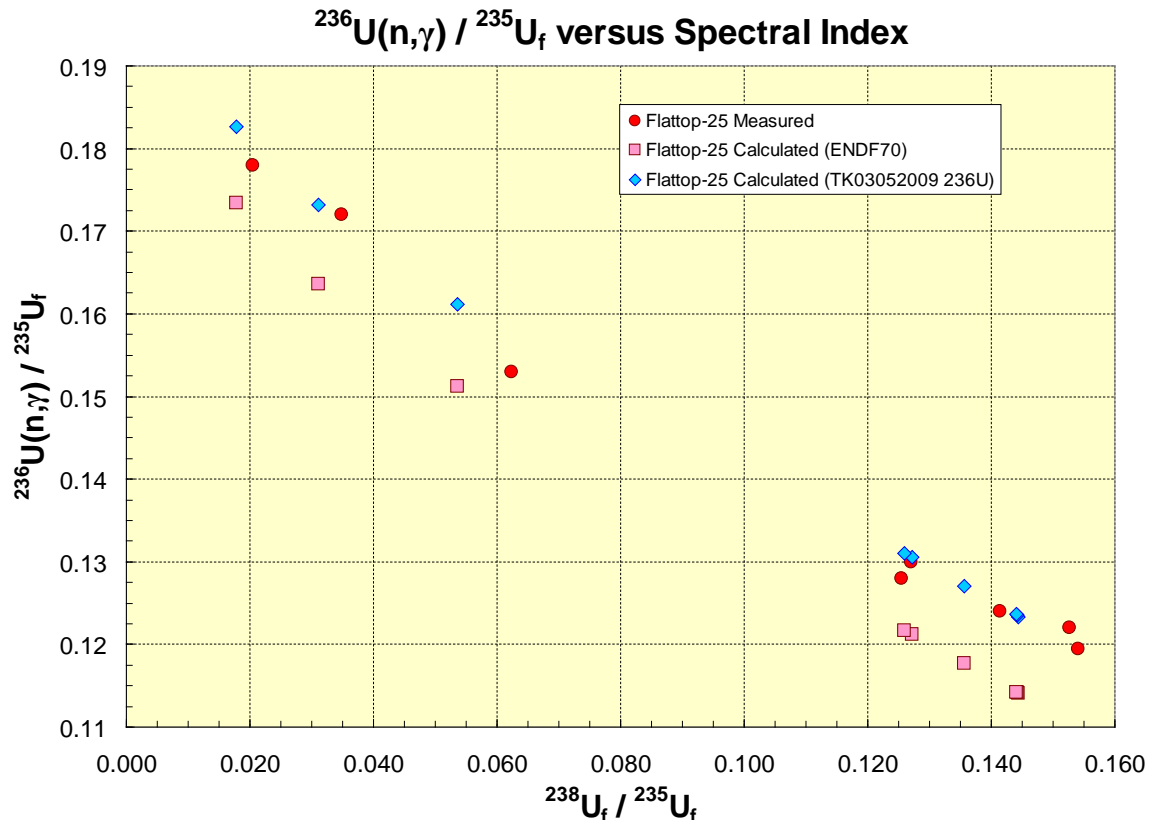
- A topic for later this session

Actinides

■ $^{236,237}\text{U}$, ^{237}Np , ^{240}Pu , ^{241}Am

- Changes to basic data.
 - ^{236}U : increased capture cross section above ~ 100 keV.
 - ^{237}U : adopt JENDL-3.3 resonance parameters and revise fission cross section below ~ 1 MeV.
 - ^{237}Np : revisions to $(n,2n)$, particularly near threshold.
 - Good reaction rate agreement with unpublished Big-10 measurements.
 - ^{240}Pu : fission cross section revisions in the tens of keV to low MeV region.
 - ^{241}Am : small changes to fission and capture cross sections below 1 MeV.
- Nuclear data changes are within the uncertainties in the underlying microscopic data, take advantage of new model calculations and experimental data and/or borrow from more modern evaluations.
- Changes are also guided by comparison to critical assembly reaction rate ratio measurements.

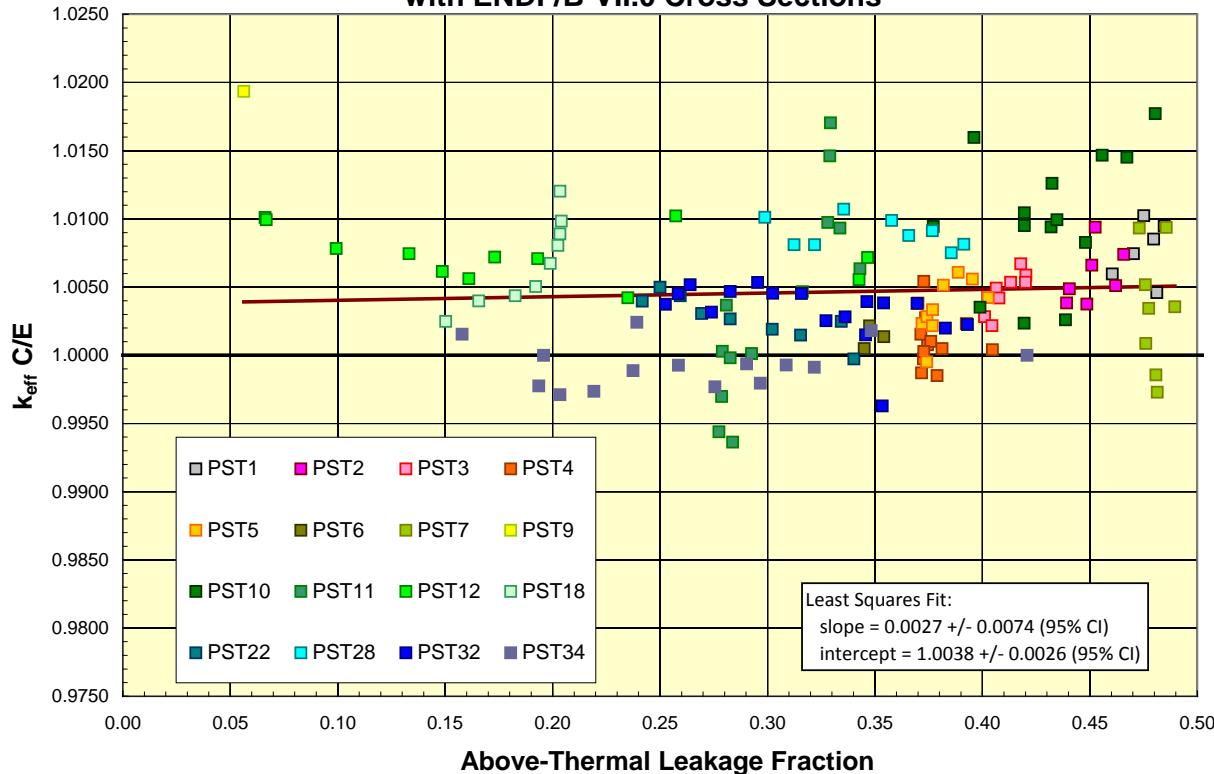
Actinides



- Calculated ^{236}U capture is biased high with ENDF/B-VII.0
- Revised evaluation from TK has eliminated this bias.

^{239}Pu

PU-SOL-THERM Benchmark Eigenvalues
with ENDF/B-VII.0 Cross Sections



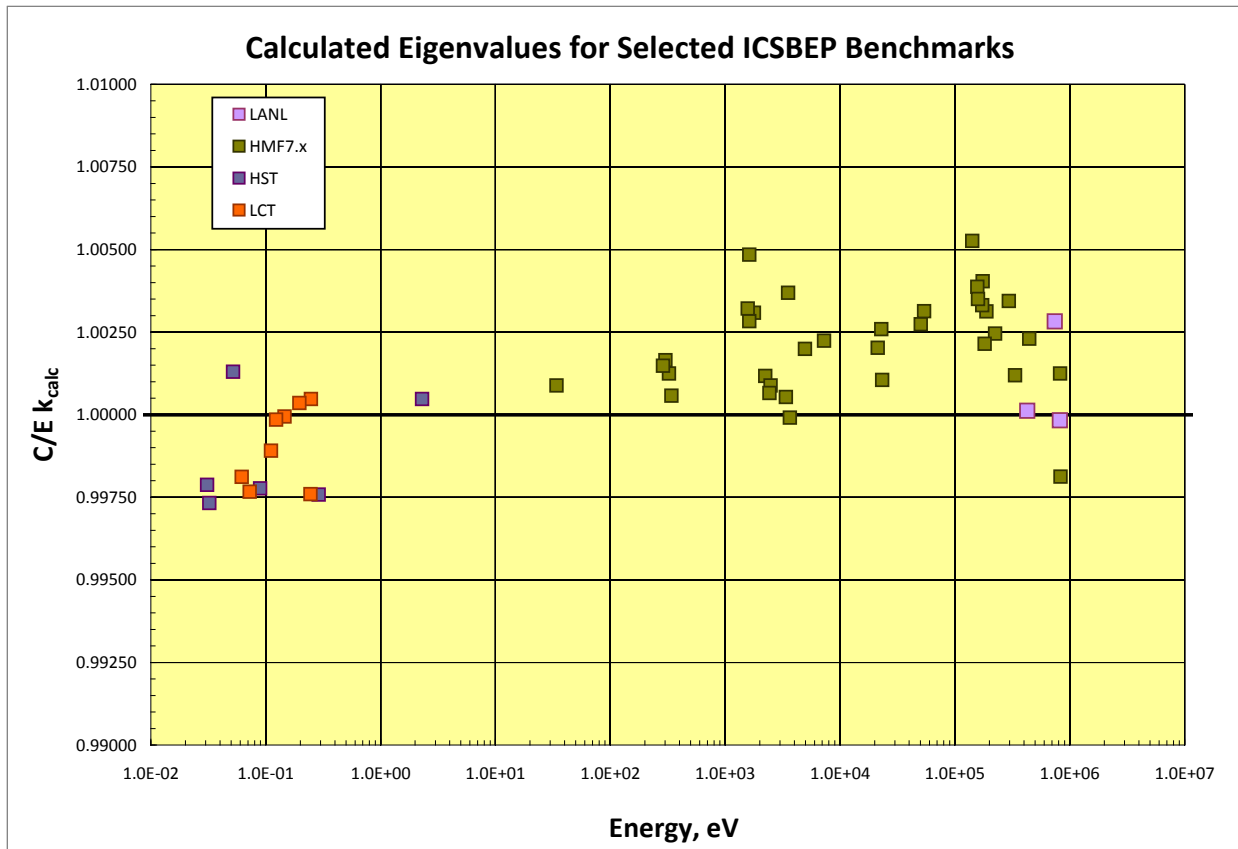
- Calculated ^{239}Pu thermal solution eigenvalues are biased high by ~500 pcm.
- Benchmark suite contains ~150 experimental configurations.
- No obvious trend (vs. ATLF in this figure).

^{239}Pu

■ Data Testing methodology

- Select representative Pu solution benchmarks that span the parameter space
 - Parameters include:
 - Above-Thermal Leakage Fraction (ATLF)
 - Above-Thermal Fission Fraction (ATFF)
 - Atom Percent ^{239}Pu (a/o ^{239}Pu)
 - Grams Pu per liter (g/l)
- Re-calculate eigenvalues for this subset of benchmarks with revised ^{239}Pu data sets.
 - Potential revisions include
 - Refined resolved resonance parameters (ORNL/Leal)
 - Revised prompt fission neutron spectrum (Maslov, Talou, refined Madland-Nix(?), ...)
- This work is at an early stage.
 - Initial result with latest Maslov spectrum seems to raise calculated eigenvalues.
 - Inclusion of Leal's latest RR parameters reduces the calculated eigenvalues to near unity (☺), but some indication of trends.
- **Much more to do here!**

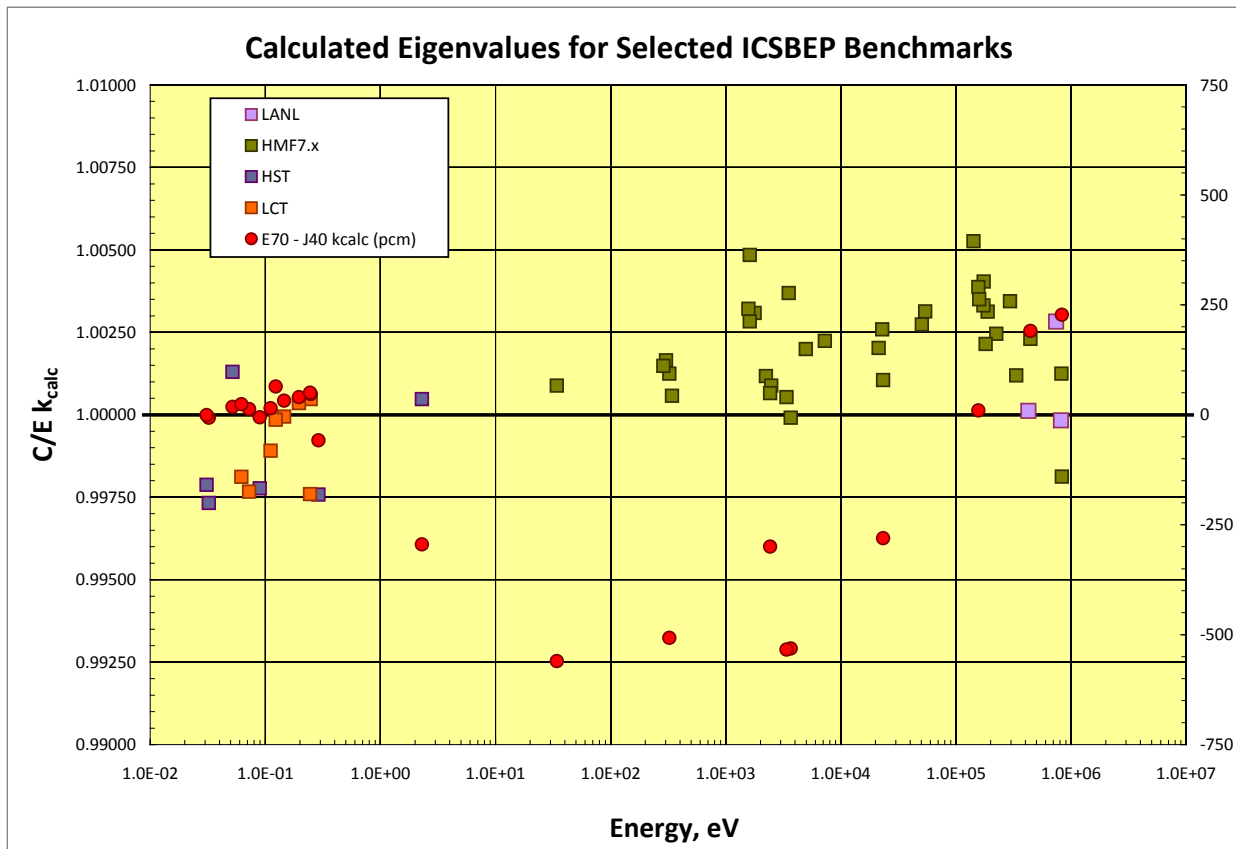
ENDF/B-VII.0 & JENDL-4.0 ²³⁵U



■ A selection of Fast, Thermal and Intermediate Spectrum Benchmarks

- Godiva, Flattop-25, Big-10
- HST's for min and max average energy
- LCT-6 (Japan) & LCT-7 (France)
- HMF7 (ORNL)

ENDF/B-VII.0 & JENDL-4.0 ²³⁵U



- Red Circles illustrate (E70 – J40) k_{calc} difference (scale given on right side axis).
- Relatively smooth parabolic trend is observed.
- JENDL-40 is more reactive for intermediate spectrum energies.