

ENDF/B-VII Beta 2 Covariance Data Testing

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Create Covariance Matrices with PUFF-IV and ERRORJ

Using ENDF-VII-B2 library files containing File 31, 32 and/or File 33 we calculated all possible covariance matrices using PUFF-IV and NJOY/ERRORJ

Calculation was done at T=0 K for PUFF-IV and T=1.1 K for NJOY/ERRRORJ

Covariance matrices with equal MAT and MT values

	1	2	5	17	18	51	102	452	456	801	851	852	853	854	855	File 32 exists
Au 197											MA.					The second second
Bi 209																
Th 232									110	4						
U 235																Matrix exists
U 238						-	Ш							-11-11		

Several explicit and implicit cross matrices are defined and are calculated by PUFF-IV and ERRORJ



Covariance matrices with equal MAT and MT values

	1	2	4	5	16	22	28	51	102	103	105	107	800	801
Li 6														
Li 7				44				41-				4		
B 10														
С														
F 19														
Na 23											-		170	
Ti 48														
٧			7									4		
Co 59														
Ni 58											11			
Nb 93														
Gd 152													100	
Gd 153														
Gd 154														
Gd 155														
Gd 156														
Gd 157														
Gd 158								0					100	
<i>G</i> d 160														

File 32 exists

Matrix exists

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MT₁=1, MAT₁=328

. . .

MT₂=851, MAT₂=328: 1 NI

NI: lb = 5, asymmetric

. . .

MT1=2, MAT1=328

MT2=851, MAT2=328:NI, NC

NC: Ity = 0: 1, 4, 851

NI: lb = 6

 $MT_1 = 4$, $MAT_1 = 328$

• • •

MT₂=851, MAT₂=328: 1 NI

NI: lb = 5, asymmetric

 $MT_1 = 102$, $MAT_1 = 328$

. . .

MT₂=851, MAT₂=328: 1 NI

NI: lb = 5, asymmetric

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⁷Li, MAT=328

Missing lumped reaction covariance data referenced in evaluation

PUFF-IV

- Lists all possible covariance matrices
- •Issues warning that some cross section data are not available
- Skips calculation of matrices that cannot be evaluated

ERRORJ

- Aborts since constituents of lumped reaction not found.
- Runs through if all references to lumped reaction 851 are deleted from ENDF file.

Missing, (present in ENDF/B-VI)

MT=16, MAT=328, component of MT=851

MT=24, MAT=328, component of MT=851



²³²Th, MAT=9040

²³²Th File 31 content

MT₁=452, MAT₁=9040 MT₂=452, MAT₂=9040: 2 NI NI: lb = 1NI: lb = 5, asymmetric $MT_2 = 452$, $MAT_2 = 1380$: 1 NI NI: lb = 4 $MT_2 = 452$, $MAT_2 = 1381$: 1 NI NI: lb = 3MT₂=452, MAT₂=1395: 1 NI NI: lb = 3MT₂=456, MAT₂=1398: 1 NI NI: lb = 3 $MT_2 = 452$, $MAT_2 = 1399$: 1 NI NI: lb = 3

PUFF-IV

Lists all possible covariance matrices

Issues warning that some cross section data are not available

Skips calculation of matrices that cannot be evaluated

ENDF-V MAT

ENDF-V MAT

NJOY

Fails to skip over File 6 while executing groupr.

Runs through once File 6 is deleted (not used for File 31, 32 or File 33 covariance calculation)

ENDF-V MAT

ENDF-V MAT

ENDF-V MAT

ERRORJ

Fails to skip over File 6 while executing groupr if processing File 31.

Runs through once File 6 is deleted (not used for File 31 covariance calculation).

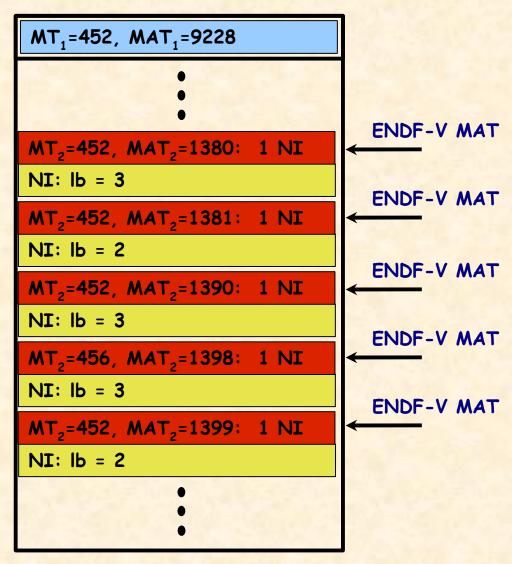
Skips calculation of matrices that cannot be evaluated

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²³⁵U, MAT=9228

²³⁵U File 31 content



PUFF-IV

- Lists all possible covariance matrices
- Issues warning that some cross section data are not available
- Skips calculation of matrices that cannot be evaluated

ERRORJ

Skips calculation of matrices that cannot be evaluated

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ENDF-VII-B2 standard files

Standard files contain standard reaction covariance matrices. The same matrices are expected to be present in the neutron library files.

	Neutron library file	Standard file	
6Li	<mt=105;mt=105></mt=105;mt=105>	<mt=105;mt=105></mt=105;mt=105>	√
C		<mt=2;mt=2></mt=2;mt=2>	×
¹⁹⁷ Au	<mt=1;mt=1></mt=1;mt=1>	<mt=102;mt=102></mt=102;mt=102>	×
235	<mt=452; mt="452"> <mt=456; mt="456"></mt=456;></mt=452;>	<mt=18;mt=18></mt=18;mt=18>	×
²³⁸ U		<mt=18;mt=18></mt=18;mt=18>	×



ENDF MAT with problems in NJOY/ERRORJ

· 6Li

ERRORJ constructs a union grid with too many energy boundaries and energy boundaries out of order.

This was fixed in ERRORR. If the same fix is applied in ERRORJ, the covariance data can be calculated

· C

NJOY fails to skip over File 12. The covariance matrices can be calculated if File 12 is deleted from the ENDF file.

ENDF file contains only a File 32 and no File 33. ERRORJ only processes File 32 if a File 33 is present.

